

# Summary of ITPA Topical Group on MHD Stability

Padova, Italy

Oct 4-7, 2011

## US Participants:

### Theory

S. Jardin  
F. Waelbroeck  
J. Harris(R)  
V. Izzo

### Experiment

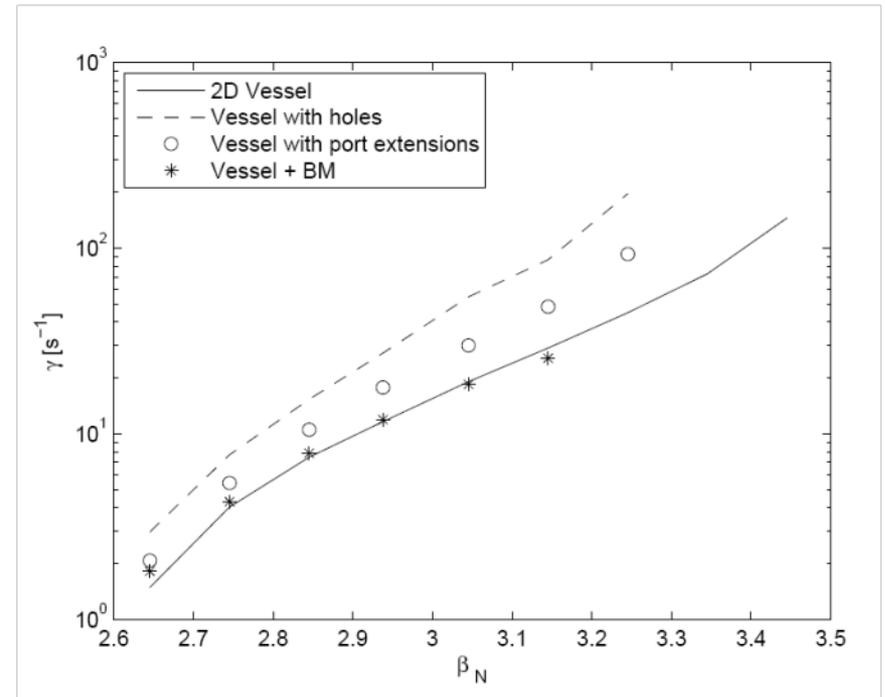
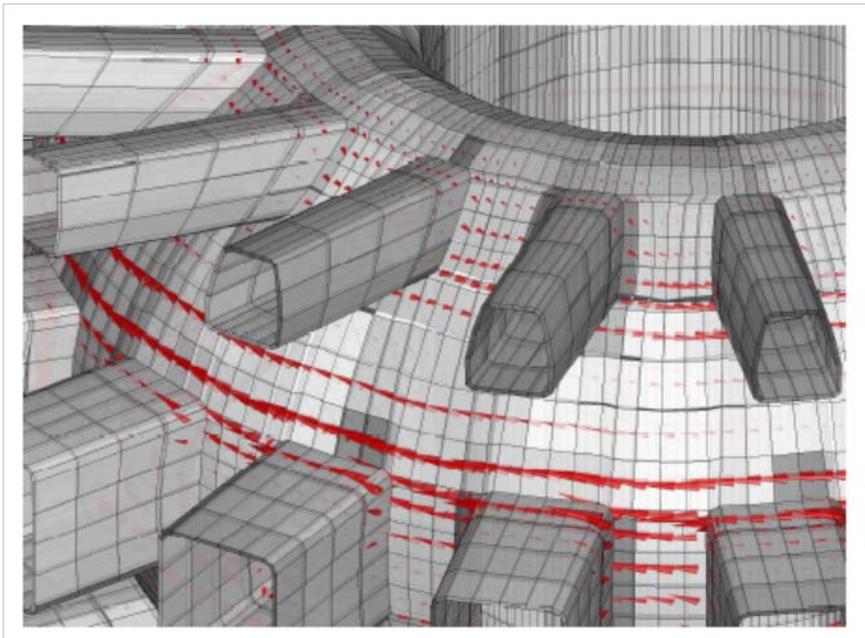
E. Strait  
R. Granetz  
S. Sabbagh  
M. Okabayashi  
J. K. Park  
R. LaHaye  
N. Eidietis (R)  
J. Wesley (R)  
D. Humphreys (R)  
R. Buttery  
A. K. Sen

# TOPICS

- 3D Physics
- Error Fields
- Resistive Wall Mode Stability and Control
- Disruption Modelling and Experiments
- Disruption Avoidance and Control
- Disruption Mitigation
- Axisymmetric Control

# 3D Physics: 3D electromagnetic analysis of RWMs (Villone)

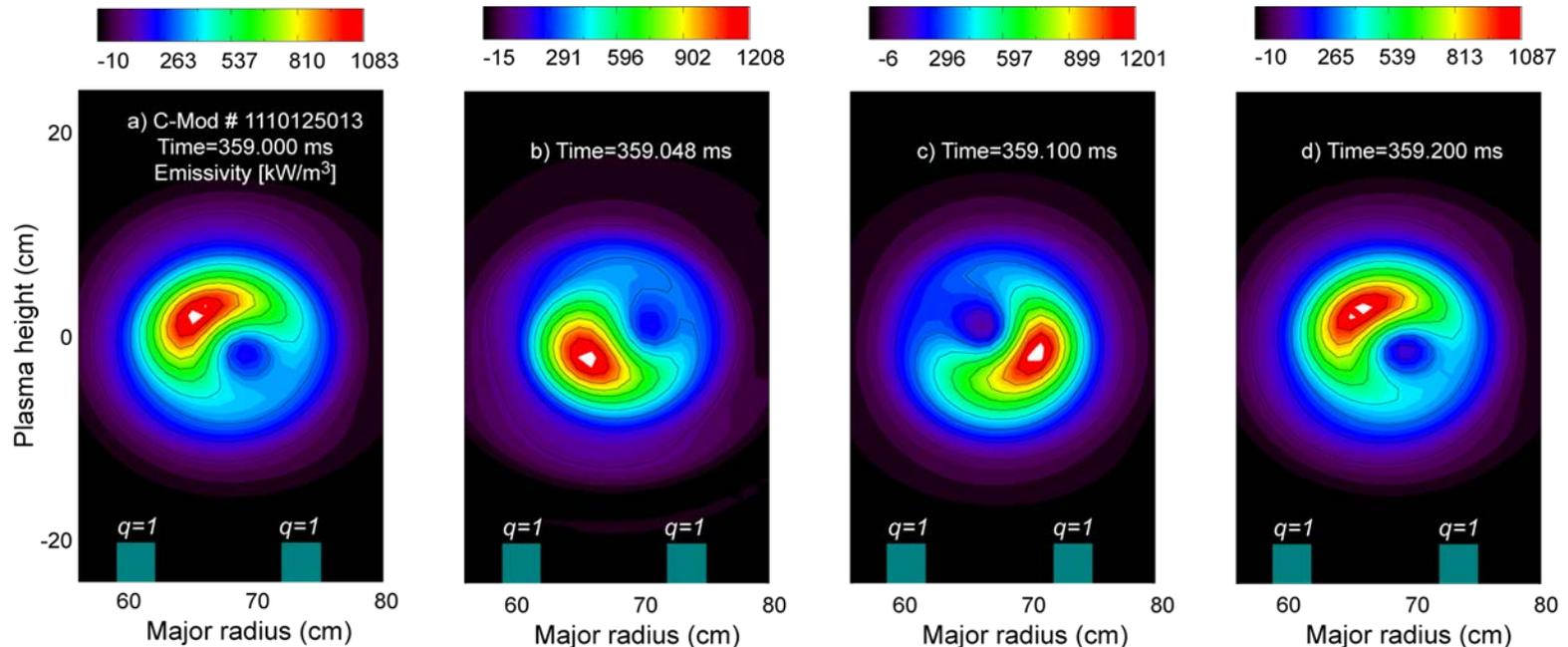
MARS code has been coupled to a full 3D model of the ITER vacuum vessel



- CarMA code combines EM structures code with MARS linear plasma code
- Being used to compute RWM, n=0 mode, active control, etc.
- Relatively small effects over 2D structure model so far.
- **Can we couple NIMROD M3D-C<sup>1</sup> to 3D structure model?**

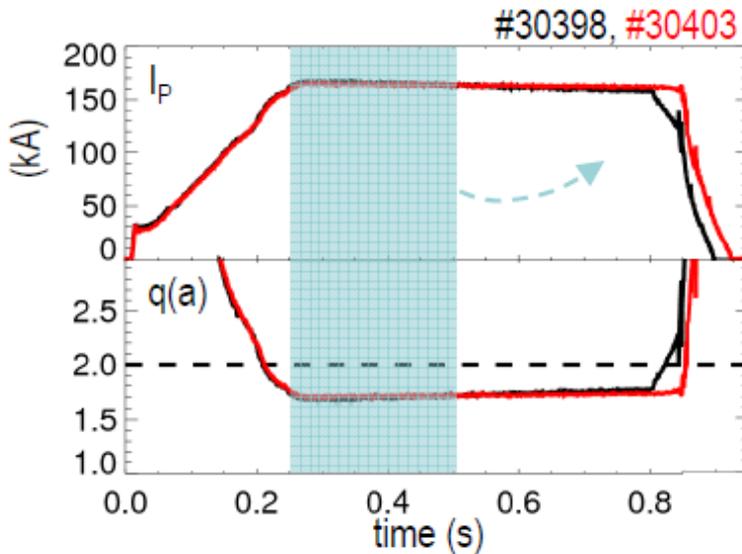
# 3D Physics: Formation and stability of m=1 impurity-induced snakes in Alcator C-Mod

(Granetz, Delgado-Aparicio, Sugiyama)

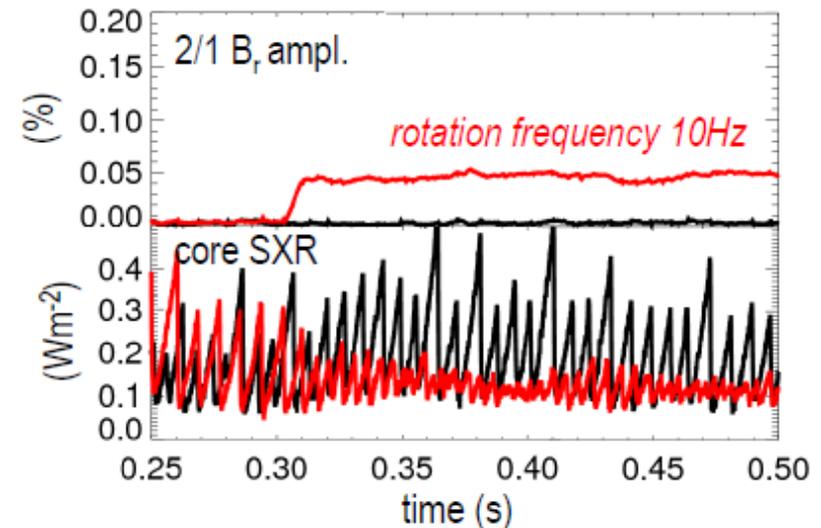


- Stationary m=1 structure in x-ray emission caused by high-Z injection
- Snake structure rotates with plasma
- Snake structure survives many sawtooth crashes !
- **Sugiyama to model?**

# 3D Physics: Sawtooth tailoring by the application of external 3D magnetic fields in RFX-mod Ohmic tokamak plasmas (Piovesan)



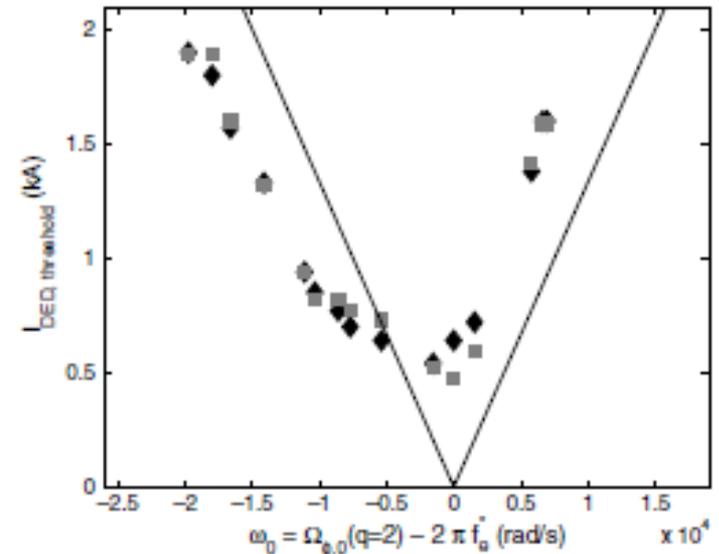
*3D fields to keep 2/1 at low amplitude*



- The RFP “RFX” can run as a Ohmically heated tokamak
- The magnetic feedback system can stabilize the 2/1 mode and allow  $q(a) < 2$  operation
- If the feedback system allows a 2/1 mode at low amplitude, sawtooth is suppressed
- Instead of sawtooth, obtain a stationary  $m=1$  kink deformation of the core.
- *Can we model this?*

# 3D Physics: Plasma Response to Resonant Magnetic Perturbations—Waelbroeck, et al.

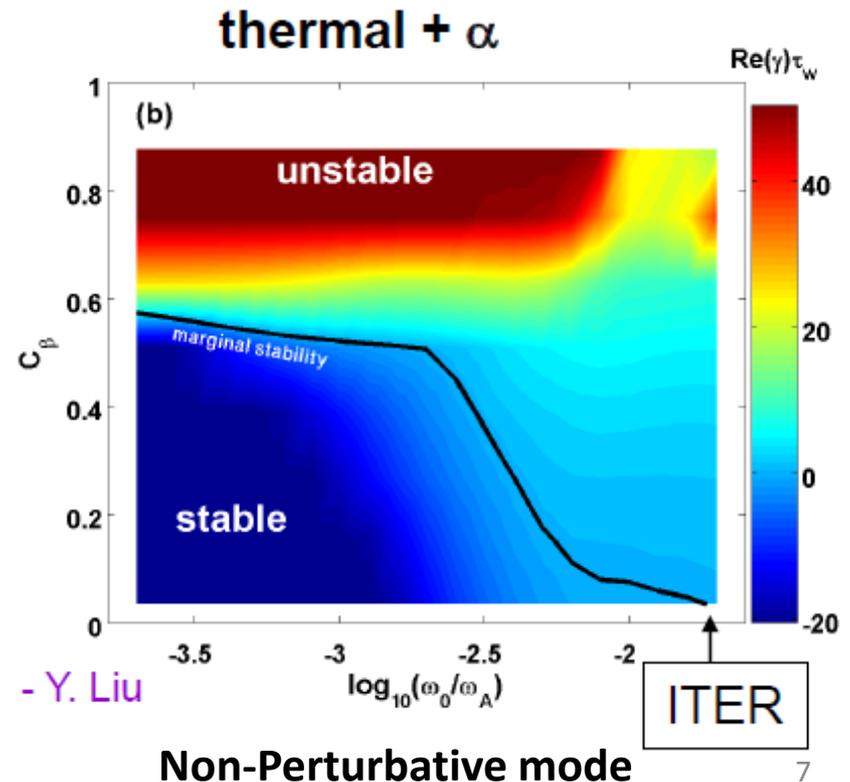
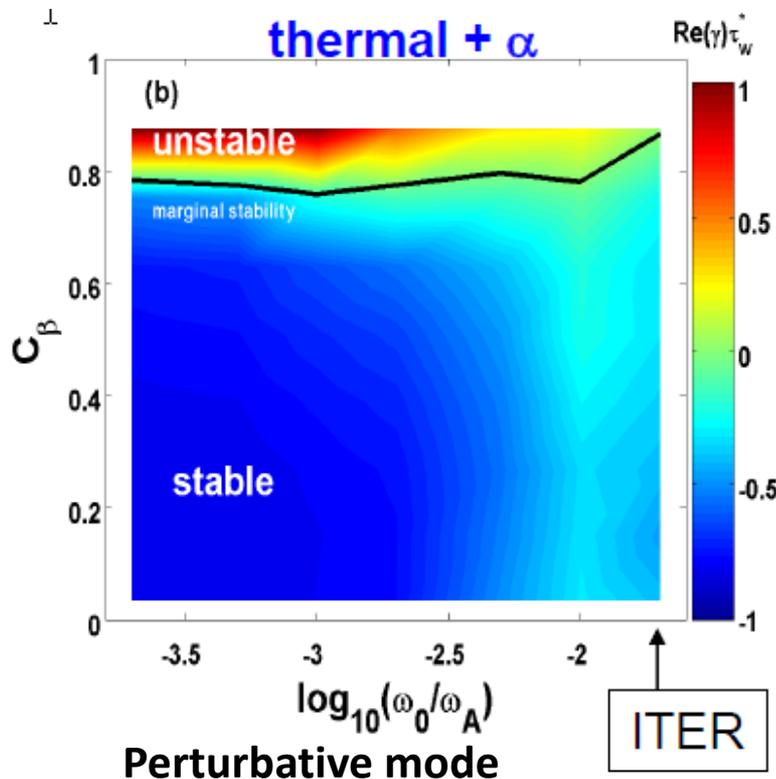
- Misc. topics on RMPs involving 11 authors
- Theory says there should be no error-field threshold if the electron rotation frequency is zero. Experimentally, there is a non-zero error-field threshold. Koslowski NF 46 (2006), DeBock NF 48 (2008)
- **Can we resolve this by modeling?**



- Suppressed RMPs give rise to particle transport: The same currents that produce the breaking force also transport particles.
- Kinetic (gyro) electrostatic simulations show turbulent breaking from islands
- Electromagnetic GYRO simulations show unlocking and screening of islands in broad agreement with fluid predictions except for the turbulent breaking

# Error Fields and RWM Control: Benchmarking RWM stability physics between codes and experiments....S. Sabbagh

- Benchmark kinetic codes: HAGIS, MARS-K, MISK in non-perturbative mode
- Predictions for ITER from MARS-K: Perturbative and non-perturbative predictions very different for low rotation. When run in self-consistent approach, shows RWM stability only at the no-wall beta limit ( $C_\beta = 0$ ) without active feedback.
- → non-perturbative modeling seems to be only relevant approach!



- Y. Liu

# Error Fields and RWM Control: Error field threshold study in NSTX high- $\beta$ plasmas – J.K. Park

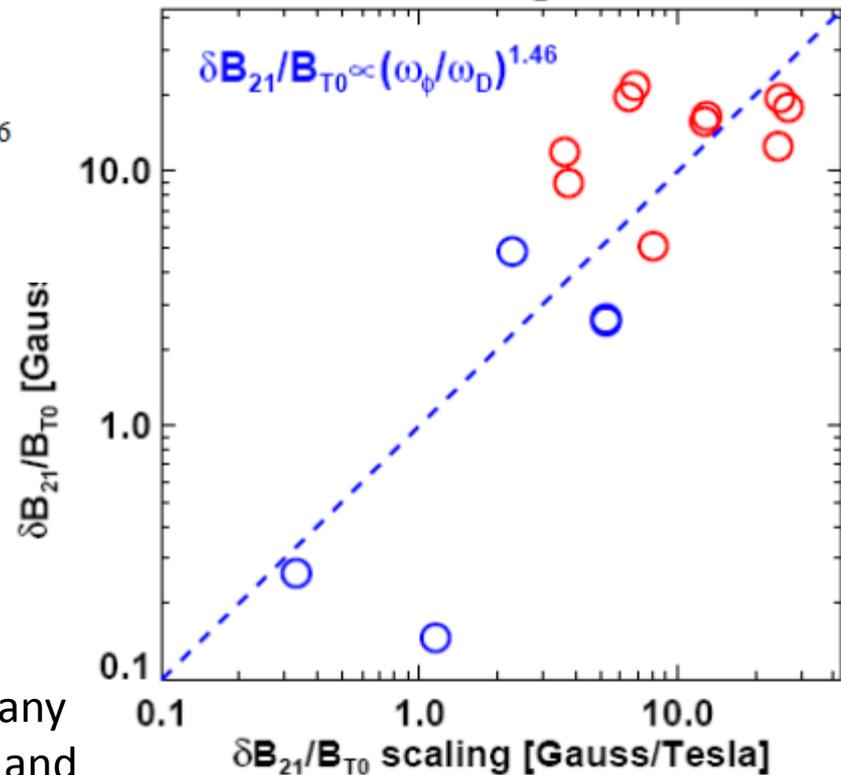
A (2,1) locked mode will appear when:

$$\frac{\delta B_{21}}{B_{T0}} \cong 2.2 \times 10^{-4} n_e^{0.60} B_{T0}^{0.45} S^{0.76} R_0^{1.74} \left( 0.22 \frac{\omega}{\omega_D} \right)^{1.46}$$

↑  
Total (2,1) field (including 3D perturbed currents in plasma)

↑  
Experimentally inferred from many NSTX experiments, both Ohmic and NB heated, some with n=3 breaking to reduce rotation.

Threshold scaling with rotation



Modeling ?

# WG-9: Criteria for Error Field Correction for ITER— R.J. Buttery, et al

- Present strategy for canceling error field in ITER is to assume dominant coupling is through least stable ideal mode ... using ideal MHD code IPEC
- But, IPEC over predicts benefit of reducing  $q=2$  resonant error field in DIII
- Need to look at origins of residual corrected error field results.
  - Error fields brake rotation to trigger tearing

- Total possible error:  $\delta B/B \sim 2.8 \times 10^{-4}$
- Cf predicted threshold  $\delta B/B \sim 1.3 \times 10^{-4}$



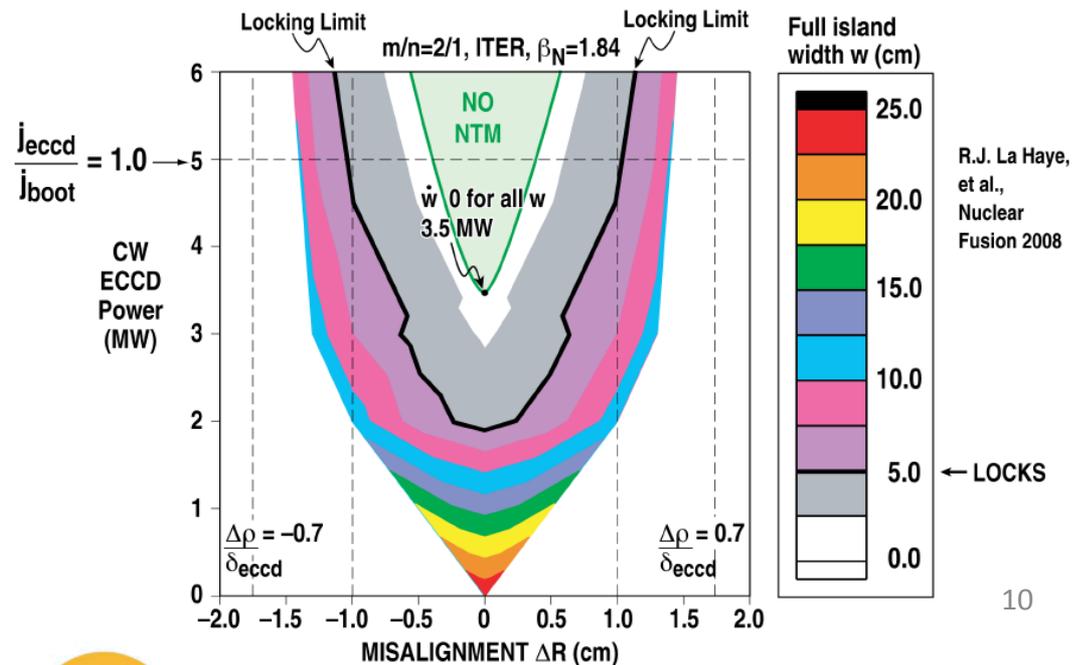
Need EF correction.  
However, correction benefit depends on shape of EF and coils.

- **Need for further modeling** interpretation to scale error field correction to ITER
  - Experiments have quantified scale of effects
  - But not which precise braking mechanisms are going on
  - Consider resistive response ? M3D-C<sup>1</sup> / MARS

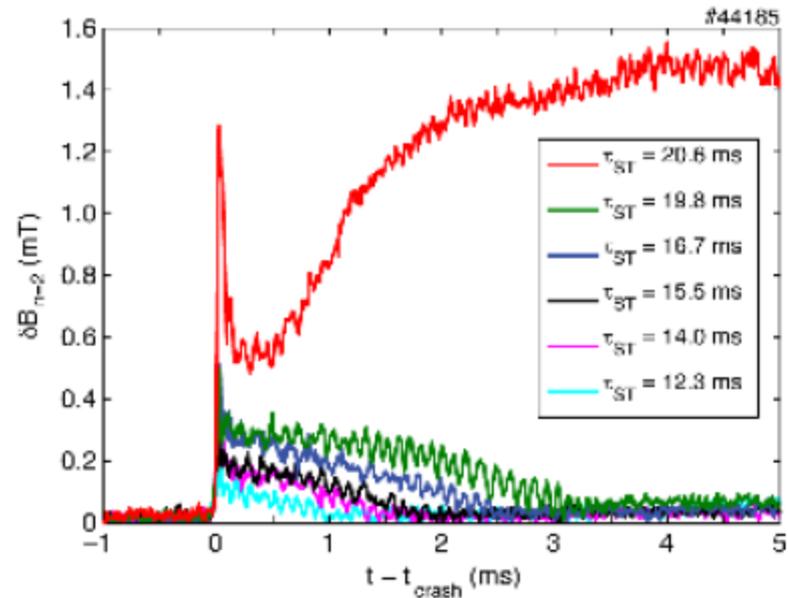
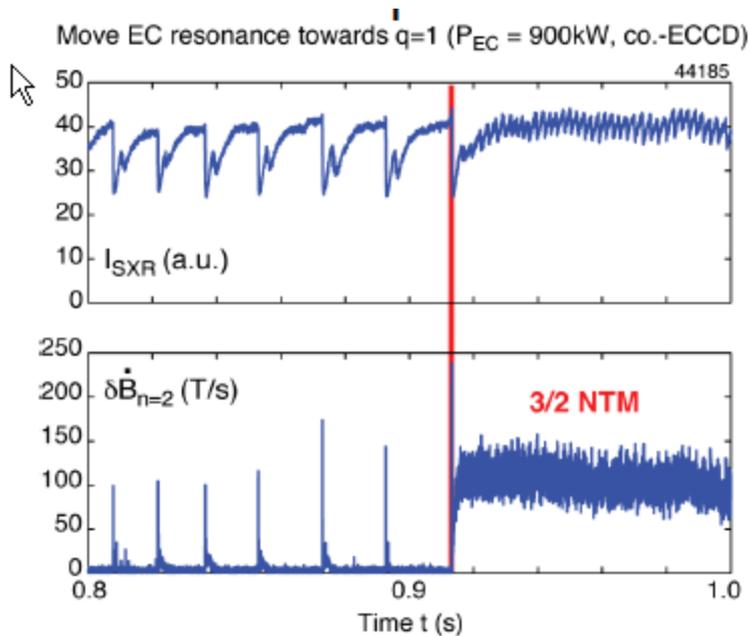
# Progress on ITER NTM Control Assessment -- R. LaHaye

- NTMs are the principal MHD stability problem for ITER
- Uncontrolled growth is predicted to lead to confinement deterioration and possibly disruption
- Rotating island induces “eddy” currents in vacuum vessel wall
  - Exerts drag at island surface
  - Can stop plasma rotation...locks, lose H-mode, disruption
- ITER relies on successful ECCD stabilization of NTMs

- This graph is for un-modulated ECCD
- Slightly less ECCD power is required for modulated power
- To what extent can NIMROD reproduce this?



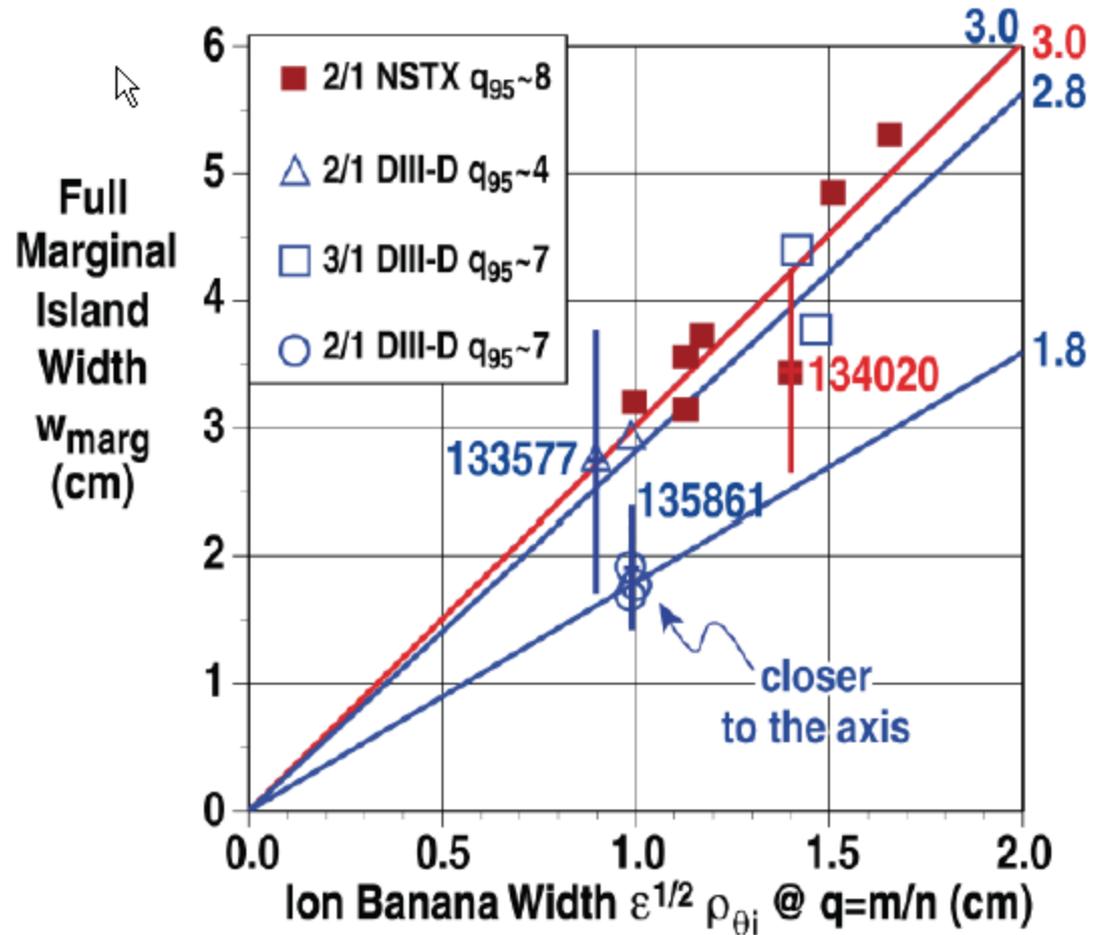
# MDC-5: Sawtooth control methods for NTM suppression-- Chapman



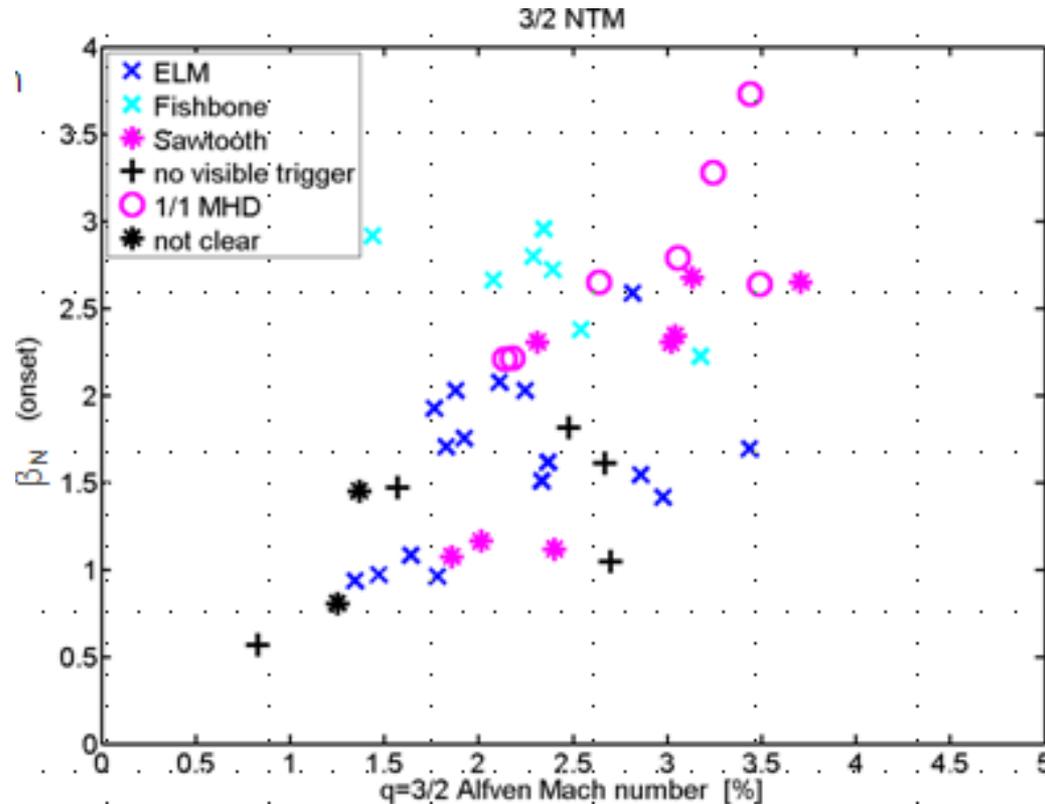
- Increase ST period by moving co-ECCD from axis towards the  $q=1$  surface
- ST with period over 20ms generates a 3/2 NTM within 1 ms
- plasmas with larger safety factor tolerate longer sawteeth
- Experiments on TCV, DIII-D, ASDEX, JET (ICRH), FTU, Tore Supra
- **This is an ideal modeling opportunity!**

# MDC-4: NTM aspect ratio comparison -- Maraschek

- Comparisons with MAST, AUG, DIII-D, NSTX
- Empirically, marginal island width for NTM onset (required seed island size) is 2~3 time ion banana width
- Can we reproduce this by adding neoclassical effects to NIMROD/M3D-C<sup>1</sup>?



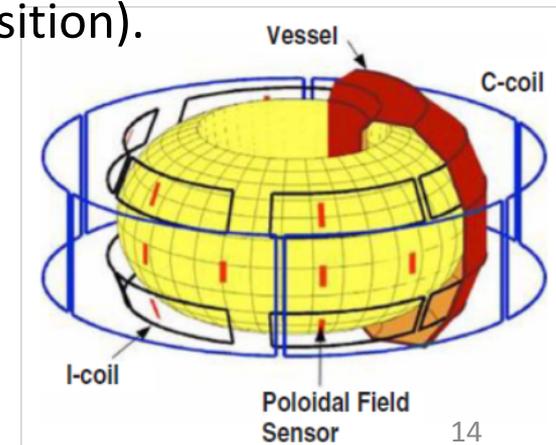
# MDC-14: Rotation effects on NTMs -- Buttery



- Higher rotation velocities allow higher  $\beta_N$  before NTM sets in.
- This data is from AUG.
- **No good theory to explain  $\beta_N$  dependence of onset of NTM on rotation:**
- seeding or underlying tearing stability?

# Stabilization of disruptive locked modes by ECCD and Magnetic Perturbations -- Volpe

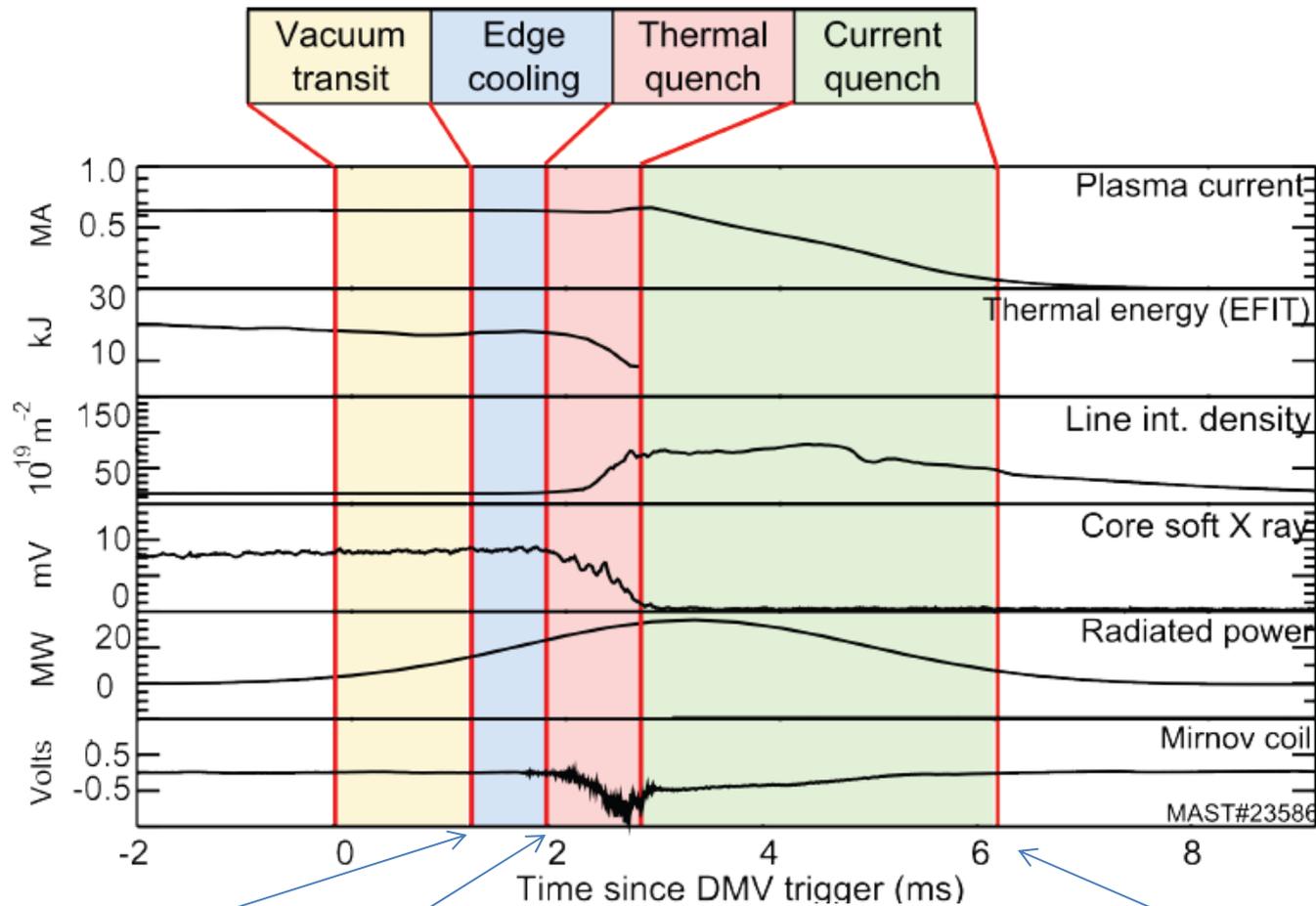
- Toroidal phase of locking was controlled by magnetic perturbations. Locked mode was rotated by RMP to a position where it could be stabilized by ECCD
- Locked Modes were completely stabilized by ECCD
- Unlocking by NBI torque was facilitated by ECCD mitigation
- Mode rotation was sustained by RMPs rotating at 20Hz, and mode stabilized by ECCD modulated at 20Hz (O-point deposition).
- **Modeling?**



# Disruption Mitigation and RE Generation (13 talks)

- All describing experimental results except V. Izzo
- Unified picture: (Pautasso)
  - gas ionizes at plasma edge
  - only (small) fraction of impinging neutrals remains confined in plasma
  - fast diffusion along magnetic field lines slowed down by low temp.
  - slower perp. diffusion and penetration of gas from edge to  $q=2$
  - development of large-scale MHD modes, dominated by  $m=2/n=1$
  - magnetic field stochastization allows for fast diffusion of particles and energy respectively into and out of plasma core ... thermal quench
  - subsequent current quench causes vessel forces and RE generation
- A code which models all aspects of MGI experiments and can extrapolate them to ITER does not exist
  - NIMROD (and M3D?) should be developed further for this purpose

# Disruption Mitigation on MAST-Thornton, et al.



Free expansion  
in vacuum

Thermal quench  
occurs when  $q=2$   
surface is cooled

Mitigation reduces divertor  
energy loading to about 40%  
of  $W_{\text{tot}}$

Current quench time  
does not change  
much due to MGI

# MDC-17: Active Disruption Avoidance -- Maraschek

- Magnetic control of toroidal position of locked mode + ECCD in DIII-D
- Application of NBI to unlock mode by imparting torque to plasma
- Disruption avoidance (or delay) with ECRH at  $q=2$  surface in AUG
- Avoidance of DL disruption by monitoring  $D_\alpha$  in/out ratios, removal of gas puff, and adding NBI
- **Modeling?**

# Meeting summaries are at:

<https://portal.iter.org/departments/POP/ITPA/MHD>

(or, contact [jardin@pppl.gov](mailto:jardin@pppl.gov) .... I can send you a copy of the agenda, and I have copies of all viewgraphs)

## Next meetings:

**March 5-9, 2012 in Toki, Japan** (in conjunction with 16<sup>th</sup> US-Japan Workshop on active MHD control. )

THEME: Effect of 3D Magnetic Fields on MHD Equilibrium and Stability: toward optimum control of toroidal plasmas. See:  
<http://dg1.nifs.ac.jp/itpa2012>

**Oct 15-17, 2012 in San Diego** (tentative)