# Progress on "flux pumping", ELMs, disruptions, and VDEs

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**CEMM Meeting** 













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GA	Lao/Lyons	RMP and Kinetic MHD in M3D-C1
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# Flux Pumping\*

\*Invited talk, Tuesday 11:00am Jardin, Ferraro, Krebs, PRL (Nov 2015)

# $\beta \equiv \mu_0 p/B^2 = 2\%$ behavior much different from low $\beta$



- At low-β, plasma kinetic energy (and T<sub>e0</sub> and q<sub>0</sub>) undergo periodic oscillations where current peaks, reconnection occurs and process repeats (sawteeth)
- At 2% β, plasma goes into a stationary state with large helical flow patterns and ultralow magnetic shear with q=1 in center



# of poloidal transits

# Ultra-flat q profile drives interchange instability

Plotted is U on one toroidal plane ( $\phi$ =0) from a 3D simulation where:  $V_{1,1} = R^2 \nabla U \times \nabla \phi$ 

> Compare with the unstable eigenfunction found in [1]  $U(r,\theta,\varphi) = U_0 r [1 - (r/r_1)^2] \sin(\theta - \varphi)$



Shape of stationary nonlinear code velocity stream function agrees well with linear eigenfunction.

Flow produces electric potential:

$$\nabla \Phi \simeq \mathbf{V} \times \mathbf{B} \simeq F \nabla U$$
$$F \equiv RB_T$$

[1] Hastie and Hender, NF 28 (1988) p. 585 "Toroidal internal kink stability in tokamaks with ultra flat q profiles"

# $B_{1,1} \bullet \nabla \Phi_{1,1}$ produces dynamo voltage that sustains configuration

$$\left(\mathbf{B}\boldsymbol{\bullet}\nabla\Phi\right)_{3D_n=0}-\left(\mathbf{B}\boldsymbol{\bullet}\nabla\Phi\right)_{2D}$$



This is caused by the perturbed n=1 magnetic field acting on the perturbed n=1 potential, driven by the interchange mode

#### *How* can $\mathbf{B} \cdot \nabla \Phi$ have a non-zero toroidal average in a volume?

Now suppose  $\tilde{\mathbf{B}}$  is a small (1,1) field component resonant with  $\Phi$ :

$$\mathbf{B} = \mathbf{B}^{\mathbf{0}} + \hat{\varphi} \times \nabla \psi_{1,1} \qquad \psi_{1,1} = \varepsilon r (1 - r^2) \sin(\theta - \varphi)$$



positive definite for r sufficiently small!

# No longer constrained to $\eta \langle \mathbf{J} \cdot \mathbf{B} \rangle = \text{const}$ , central regions in all 3D runs approach minimum energy Taylor State with q=1



The nonlinear drive that keeps the current from peaking gets stronger as  $q \rightarrow 1$  from above

This feedback mechanism results in an ultra-flat q-profile in center with  $q_0 = 1 + \epsilon$  (where  $\epsilon \ll 1$ )

# ELMs

# **Plasma equilibrium used in simulations**

#### At *t* ~ **4.36** s of KSTAR discharge **#7328**

• Magnetic geometry (EFIT + TEQ)



• Equilibrium profiles across mid-plane



Minwoo Kim



Close-up of adapted mesh



Kinetic energy in different toroidal harmonics vs time:

Nonlinear 84 plane run on Hopper:





# Current Density $\Delta^{\!*}\psi$





t = 360

Superposition of mid-plane profiles of current and pressure shows that instability eats away at edge and doesn't affect central region.



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Pressure contours at every 2° (toroidal angle) shows there are no grid-point to grid-point oscillations (grids are at every 360/84 = 4.28 degrees)



#### 2º (toroidal angle) scan continued



# Linear growth rates

Thermal conductivity is very stabilizing!

(ntor=8)

$\kappa_{\perp}$	κ <sub>II</sub>	γ
0	0	.050
0	1	.032
1. E-7	1	.031
1. E-6	1	.020
1. E-5	1	Stable!

# Disruption caused by current rampdown

# Unique Class of Major Disruptions Identified in NSTX

- Recipe:
  - Generate a stable low(er) q95 discharge.
  - Run it to the current limit of the OH coil.
  - Ramp the OH coil back to zero, applying a negative loop voltage, while leaving the heating on.
  - Watch I<sub>i</sub> increase, then disruption occurs.
- Mechanism responsible for 21 for the 22 highest W<sub>MHD</sub> disruptions in NSTX.
- Specific example in the general area of how unstable current profiles lead to catastrophic instability





shot 129922 Time 860 ms  $\begin{array}{l} \mathsf{I_{P}} & \sim 1.1 \text{ MA} \\ \mathsf{q_{0}} & \sim 1.22 \\ \beta & \sim 6 \ \% \end{array}$ 

Te(0) = 1.14 keV V<sub>L</sub> = 0.36 Volts  $\chi$  = 1 m^2/sec

#### **Numerical Parameters:**

#### Entire domain



# $k+1 \frac{\varphi}{\varphi} \frac{k}{k}$

Triangular prism finite elements

#### 10 cm x 10 cm patch



 $S = 10^7$  (in center)

2D triangle size: 2 – 4 cm

32 and 64 toroidal planes

# Within each element, each scalar field is represented as a polynomial in $(R, \varphi, Z)$ with 72 terms. All first derivatives are continuous.

This is a challenging problem because:

- Both current diffusion (transport) and ideal MHD (stability) time scales
- Requires high resolution for high-(m,n) modes
- Heating and particle sources
- Loop voltage prescribed at computational boundary
  - Control system to keep plasma current fixed before ramp-down
  - Switch to fixed negative value at start of current ramp-down

# initial 64 plane run



- n=7,8,9,10,11,12 are most linearly unstable
- n=1,2,19,20 are nonlinearly driven
- Other modes not shown

#### 64 planes

## Toroidal derivative of pressure at several time slices



Same color scale:

First becomes unstable at very edge, then instability moves inward. Retains linear structure.

#### Voltage reversed at 1.28 ms

Becomes limited shortly after ramp-down starts. Impurity generation??

#### 64 planes

### Plasma current density at several time slices



#### 64 planes

## Plasma current density at several time slices



Different color scheme from previous viewgraph. Red and yellow are positive, blue is negative, zero is white.

## Toroidal derivative of poloidal flux at several time slices



Same color scale for all times. Same patter, just grows.

### Perturbed pressure and currents at time of saturation are very similar for 32 plane and 64 plane cases



# (approx) comparison with experiment



Experimental comparison not exact:

- Did not try and match Te profile
- Simulation used idealized V<sub>L</sub> reversal
- Did not use realistic vessel

However:

- Fair agreement for initial I<sub>P</sub> decay rate
- Fair agreement for initial  $\beta$  decay rate

Remaining issues:

- Can we reproduce current spike?
- Can we reproduce later rapid β drop?
- Need to dissipate short toroidal wavelengths in simulation
- Hyper-resistivity?
- Long running time for 40 ms

# **Future Plans**

- Improve preconditioner to allow larger time steps for runs with high toroidal resolution
- Investigate the effect of hyper-resistivity during the current ramp-down. Can we get a current spike?
- Can we reproduce the rapid thermal quench? Is impurity radiation required?
- Determine what is an acceptable current ramp-down rate to avoid rapid thermal quench? Compare with experimental result.

# VDE

# NSTX Shot 132859 $\eta_w$ =0.00025



# **NSTX Shot 132859** $\eta_w$ =0.00025



## **NSTX Shot 132859** η<sub>w</sub>=0.00025



## **NSTX Shot 132859** η<sub>w</sub>=0.00025

Disruption phase 2700 < t < 2950

 $\partial(RJ_{\varphi})/\partial\varphi$ 

Magnitude of the toroidal derivative of the toroidal current at one poloidal plane at the 5 times shown. Each color scale is adjusted to maximum range.



# **NSTX Shot 132859** η<sub>w</sub>=0.00025

Disruption phase 2700 < t < 2950

 $J_{_{arphi}}$ 

Top is magnitude of the toroidal current at one poloidal plane at the 5 times shown. Each color scale is adjusted to maximum range. Bottom is values along horizontal line of maximum current as shown.



# **NSTX Shot 132859** $\eta_w$ =0.00025

### Summary of first 3D M3D-C1 simulation of VDE in NSTX

- Plasma drifts downward with linear growth rate  $\gamma\tau_{A}$  = 0.00135 for entire drift phase: 0 < t < 2700  $\tau_{A}$
- q-profile remains fixed during this phase and plasma nearly axisymmetric
- Slow n=1 mode with  $\gamma \tau_A = 0.024$  begins to grow at t=2800  $\tau_A$  (RWM?) and this mode accelerates to  $\gamma \tau_A = 0.132$  at t=2850  $\tau_A$  (external kink?)
- Wall current is initially negative (to repel plasma) and then becomes positive as plasma current decays. Halo currents begin to form at about at t=2825  $\tau_A$  when plasma makes contact with vessel.
- n=1 mode mostly external with m ~ nq. Continues to growth in amplitude until plasma disappears
- Strong shielding currents develop once plasma makes contact with vessel. Consequence of plasma staying hot in core until the end.

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