# *Progress on "flux pumping", ELMs, disruptions, and VDEs*

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## Flux Pumping\*

\*Invited talk, Tuesday 11:00am Jardin, Ferraro, Krebs, PRL (Nov 2015)

# $\beta \equiv \mu_{0} p/B^{2}$  =2% behavior much different from low  $\beta$



- At low- $\beta$ , plasma kinetic energy (and T<sub>e0</sub> and q<sub>0</sub>) undergo periodic oscillations where current peaks, reconnection occurs and process repeats (sawteeth)
- At 2%  $\beta$ , plasma goes into a stationary state with large helical flow patterns and ultralow magnetic shear with q=1 in center



 $q = \frac{q}{\text{# of poloidal transits}}$ 

## *Ultra-flat q profile drives interchange instability*

Plotted is *U* on one toroidal plane  $(\varphi=0)$  from a 3D simulation where:  $\mathbf{V}_{1,1} = R^2 \nabla U \times \nabla \varphi$  $q = 1.01$ 

> $U(r, \theta, \varphi) = U_0 r [1 - (r/r_1)^2] \sin(\theta - \varphi)$ Compare with the unstable eigenfunction found in [1]



Shape of stationary nonlinear code velocity stream function agrees well with linear eigenfunction.

Flow produces electric potential:

$$
\nabla \Phi \simeq \mathbf{V} \times \mathbf{B} \simeq F \nabla U
$$

$$
F \equiv R B_r
$$

*T*

[1] Hastie and Hender, NF **28** (1988) p. 585 "Toroidal internal kink stability in tokamaks with ultra flat q profiles"

## $B_{1,1}$   $\bullet$   $\nabla \Phi_{1,1}$  produces dynamo voltage that *sustains configuration*

$$
\left(\mathbf{B}\bullet\nabla\Phi\right)_{3D_{-}n=0}-\left(\mathbf{B}\bullet\nabla\Phi\right)_{2D}
$$



This is caused by the perturbed n=1 magnetic field acting on the perturbed n=1 potential, driven by the interchange mode

### *How can have a non* **<sup>B</sup>** *-zero toroidal average in a volume?*

Now suppose **B** is a small  $(1,1)$  field component resonant with  $\Phi$ :  $\Phi$  :

$$
\mathbf{B} = \mathbf{B}^0 + \hat{\varphi} \times \nabla \psi_{1,1} \qquad \psi_{1,1} = \varepsilon r \left( 1 - r^2 \right) \sin \left( \theta - \varphi \right)
$$



*effective voltage along perturbed field!*

positive definite for  $r$  sufficiently small!

### *No longer constrained to*  $\,\eta\,\langle\mathbf{J}\!\cdot\!\mathbf{B}\rangle\!=\!\mathrm{const}\,$  *, central regions in all 3D runs approach minimum energy Taylor State with q=1*



The nonlinear drive that keeps the current from peaking gets stronger as  $q\rightarrow 1$  from above

This feedback mechanism results in an ultra-flat q-profile in center with  $q_0 = 1 + \varepsilon$  (where  $\varepsilon < 1$ )

# **ELMs**

# **Plasma equilibrium used in simulations**

### At *t* **~ 4.36** s of KSTAR discharge **#7328**

• Magnetic geometry  $(EFIT + TEQ)$ 



• Equilibrium profiles across mid-plane



Minwoo Kim



Close-up of adapted mesh



Kinetic energy in different toroidal harmonics vs time:

Nonlinear 84 plane run on Hopper:





### Current Density  $\Delta^*\psi$

 $t = 0$ 





 $t = 360$ 

Superposition of mid-plane profiles of current and pressure shows that instability eats away at edge and doesn't affect central region.





Pressure contours at every 2° (toroidal angle) shows there are no grid-point to grid-point oscillations (grids are at every 360/84 = 4.28 degrees)



### 2º (toroidal angle) scan continued



# Linear growth rates

Thermal conductivity is very stabilizing! (ntor=8)



# Disruption caused by current rampdown

### Unique Class of Major Disruptions Identified in NSTX

- Recipe:
	- Generate a stable low(er) q95 discharge.
	- Run it to the current limit of the OH coil.
	- Ramp the OH coil back to zero, applying a negative loop voltage, while leaving the heating on.
	- Watch I<sub>i</sub> increase, then disruption occurs.
- Mechanism responsible for 21 for the 22 highest  $W_{MHD}$  disruptions in NSTX.
- Specific example in the general area of how unstable current profiles lead to catastrophic instability



[S. Gerhardt, Nov. 2013]



shot 129922 Time 860 ms

 $I_p \sim 1.1$  MA  $q_0 \sim 1.22$  $\beta \sim 6 \%$ 

 $Te(0) = 1.14 keV$  $V<sub>L</sub>$  = 0.36 Volts  $\chi = 1$  m^2/sec

#### **Numerical Parameters:**

#### Entire domain



# $k+1$ Triangular prism

finite elements

### 10 cm x 10 cm patch



 $S = 10<sup>7</sup>$  (in center)

2D triangle size:  $2 - 4$  cm

32 and 64 toroidal planes

Within each element, each scalar field is represented as a polynomial in  $(R, \varphi, Z)$  with 72 terms. All first derivatives are continuous.

This is a challenging problem because:

- Both current diffusion (transport) and ideal MHD (stability) time scales
- Requires high resolution for high-(m,n) modes
- Heating and particle sources
- Loop voltage prescribed at computational boundary
	- Control system to keep plasma current fixed before ramp-down
	- Switch to fixed negative value at start of current ramp-down

## *initial 64 plane run*



- n=7,8,9,10,11,12 are most linearly unstable
- n=1,2,19,20 are nonlinearly driven
- Other modes not shown

#### *64 planes*

### *Toroidal derivative of pressure at several time slices*



Same color scale:

First becomes unstable at very edge, then instability moves inward. Retains linear structure.

#### Voltage reversed at 1.28 ms

Becomes limited shortly after ramp-down starts. Impurity generation??

#### *64 planes*

### *Plasma current density at several time slices*



#### *64 planes*

### *Plasma current density at several time slices*



Different color scheme from previous viewgraph. Red and yellow are positive, blue is negative, zero is white.

### *Toroidal derivative of poloidal flux at several time slices*



Same color scale for all times. Same patter, just grows.

### *Perturbed pressure and currents at time of saturation are very similar for 32 plane and 64 plane cases*



## *(approx) comparison with experiment*



Experimental comparison not exact:

- Did not try and match Te profile
- Simulation used idealized  $V<sub>L</sub>$  reversal
- Did not use realistic vessel

However:

- Fair agreement for initial  $I<sub>p</sub>$  decay rate
- Fair agreement for initial  $\beta$  decay rate

Remaining issues:

- Can we reproduce current spike?
- Can we reproduce later rapid  $\beta$  drop?
- Need to dissipate short toroidal wavelengths in simulation
- Hyper-resistivity?
- Long running time for 40 ms

## *Future Plans*

- Improve preconditioner to allow larger time steps for runs with high toroidal resolution
- Investigate the effect of hyper-resistivity during the current ramp-down. Can we get a current spike?
- Can we reproduce the rapid thermal quench? Is impurity radiation required?
- Determine what is an acceptable current ramp-down rate to avoid rapid thermal quench? Compare with experimental result.

# VDE







Disruption phase 2700 < t < 2950

 $\partial (RJ_{\varphi})/\partial \varphi$ 

Magnitude of the toroidal derivative of the toroidal current at one poloidal plane at the 5 times shown. Each color scale is adjusted to maximum range.



Disruption phase 2700 < t < 2950

 $J_{_{\scriptscriptstyle\phi}}$ 

Top is magnitude of the toroidal current at one poloidal plane at the 5 times shown. Each color scale is adjusted to maximum range. Bottom is values along horizontal line of maximum current as shown.



### **Summary of first 3D M3D-C1 simulation of VDE in NSTX**

- Plasma drifts downward with linear growth rate  $\gamma \tau_A = 0.00135$  for entire drift phase:  $0 < t < 2700 \tau_A$
- q-profile remains fixed during this phase and plasma nearly axisymmetric
- Slow n=1 mode with  $\gamma\tau_A$  = 0.024 begins to grow at t=2800  $\tau_A$  (RWM?) and this mode accelerates to  $\gamma\tau$ <sub>A</sub> = 0.132 at t=2850  $\tau$ <sub>A</sub> (external kink?)
- Wall current is initially negative (to repel plasma) and then becomes positive as plasma current decays. Halo currents begin to form at about at t=2825  $\tau_A$ when plasma makes contact with vessel.
- n=1 mode mostly external with  $m \sim nq$ . Continues to growth in amplitude until plasma disappears
- Strong shielding currents develop once plasma makes contact with vessel. Consequence of plasma staying hot in core until the end.

