NSTX Disruption Calculations with M3D

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Motivation

- A significant fraction of tokamak discharges at fusion-relevant parameters terminate in disruptions.
- As experiments are scaled up, the stored energy becomes higher, and the potential structural damage due to each disruption increases.
- Accurate quantitative prediction of the distributions of transient currents and attendant forces in conducting structures surrounding a disrupting ITER plasma is vital so that these structures can be designed to survive them.
- 3D nonlinear MHD codes with resistive wall boundary conditions are an appropriate tool for calculating currents and forces due to disruptions, but must first be validated against data from experiments such as NSTX, in which they are not catastrophic.
- VDEs are investigated first because they allow the plasma to reach the wall with most of its current, causing the greatest potential damage.

NSTX XP833 (2010): Halo current dependencies on I_p/q_{95} , vertical velocity, and halo resistance

Reference shot without forced disruption drive, based on 129416:



Shot 132859, with deliberately misadjusted vertical field control, terminates in VDE:



Layout of NSTX halo current diagnostics



Halo current is inferred from transient TF measurements under several divertor tiles and plates at about six toroidal locations. Transient vessel forces are not measured.

Figure reproduced from S.P. Gerhardt, J. Menard, S. Sabbagh and F. Scotti, Nucl. Fusion 52 (2012).

Meshing the NSTX Vessel

NSTX Vessel Model interpolated to TSC 2.0 cm grid



Toroidally Asymmetric Halo Current Figures of Merit

Toroidal peaking factor (TPF): If there are N poloidal planes, *j*=1,2,3,...,N, then

$$TPF = \frac{\operatorname{Max}_{j=1}^{N} \left\{ \oint_{\Gamma_{j}} R \big| \mathbf{J} \cdot \hat{n} \big| dl \right\}}{\frac{1}{N} \sum_{j=1}^{N} \left\{ \oint_{\Gamma_{j}} R \big| \mathbf{J} \cdot \hat{n} \big| dl \right\}}$$

Halo fraction (HF):

$$HF \equiv \frac{\frac{2\pi}{N} \sum_{j=1}^{N} \left\{ \oint_{\Gamma_{j}} R \big| \mathbf{J} \cdot \hat{n} \big| dl \right\}}{I_{p0}},$$

where I_{p0} is the total plasma current in the initial equilibrium.



Figure reproduced from S.P. Gerhardt, J. Menard, S. Sabbagh and F. Scotti, Nucl. Fusion 52 (2012).

Low-plasma-resistivity simulation parameters

Plasma resistivity on axis* η_0 =S ⁻¹	5 × 10 ⁻⁶
$\eta_{ m vacuum}$ / η_0	12000
$\eta_{ m wall}$ / $\eta_{ m 0}$	10000 ($\tau_w / \tau_A = 20$)
Prandtl number μ / η_0	20
Perpendicular heat conduction κ_{ot} / η_0	2
Effective parallel heat conduction v_{Te} / v_{A}	2
Density evolution	Off (uniform, constant)
Size of initial <i>n</i> =1 perturbation	5 × 10 ⁻³
Number of toroidal modes	5 (16 poloidal planes)

Low-resistivity Snapshots



Initial VDE-unstable equilibrium $q_0 \approx 1$

Plasma is displaced downward, *n*=1 instability is not in evidence. Confinement is lost, heat deposited in divertor region.

Time History



Halo Current Distribution at Peak

t = 109.834



High-plasma-resistivity parameters

Plasma resistivity on axis* η_0 =S ⁻¹	5 × 10 ⁻⁴
$\eta_{ m vacuum}$ / η_0	120
$\eta_{ m wall}$ / $\eta_{ m 0}$	100 ($\tau_w / \tau_A = 20$)
Prandtl number μ / η_0	0.2
Perpendicular heat conduction κ_{\perp} / η_0	0.02
Effective parallel heat conduction v_{Te} / v_{A}	2
Density evolution	Off (uniform, constant)
Size of initial <i>n</i> =1 perturbation	5 × 10 ⁻³
Number of toroidal modes	5 (16 poloidal planes)

High-resistivity Snapshots





1.5 Pseudocolor Pseude Var: p 0.04254 1.0 0.03190 0.5 0.02125 0.0 N.0.01061 -3.630e-05 Max: 0.04254 Min: -3.630e-05 -1.0 -1.5 0.2 0.4 0.6 0.8 1.0 1.2 1.4 1.6 Major Radius

 $t = 86.95 \tau_{A}$

Initial VDE-unstable equilibrium

Higher-*n* instabilities disperse heat before significant vertical displacement occurs.

Confinement is lost rapidly; heat deposited on inboard wall.

Time History



Halo Current Distribution at Peak

t = 59.434







n=1 component is about half as large as *n*=0.

Worst-case scenario: intermediate resistivity

Plasma resistivity on axis* η_0 =S ⁻¹	2 × 10 ⁻⁵
$\eta_{ m vacuum}$ / η_0	3000
$\eta_{ m wall}$ / $\eta_{ m 0}$	2500 (τ_w / τ_A =20)
Prandtl number μ / η_0	5
Perpendicular heat conduction κ_{ot} / η_0	0.5
Effective parallel heat conduction v_{Te} / v_{A}	2
Density evolution	Off (uniform, constant)
Size of initial <i>n</i> =1 perturbation	5 × 10 ⁻³
Number of toroidal modes	5 (16 poloidal planes)

Snapshots

t = 0.0



Initial VDE-unstable equilibrium



n=1 instability is concurrentwith energetic stage ofvertical displacement event.

 $t = 148.68 \tau_{A}$



Confinement is lost, heat deposited in divertor region.

Time History



Halo Current Distribution at Peak

t = 122.854



Resistivity Scaling Results



n=1 mode appears to be resistive



Actual plasma has $\eta_{wall} >> \eta_0$, so the observed MHD mode is likely ideal and destabilized by scrape-off of high-q surfaces during VDE.

New Case: restarted from t=79.69 of low-resistivity case above

Plasma resistivity on axis* η_0 =S ⁻¹	5 × 10 ⁻⁶
$\eta_{ m vacuum}$ / η_0	12000
$\eta_{ m wall}$ / $\eta_{ m 0}$	100 ($\tau_w / \tau_A = 2000$)
Prandtl number μ / η_0	20
Perpendicular heat conduction κ_{ot} / η_0	2
Effective parallel heat conduction v_{Te} / v_{A}	2
Density evolution	Off (uniform, constant)
Size of initial <i>n</i> =1 perturbation	5 × 10 ⁻³
Number of toroidal modes	5 (16 poloidal planes)

Snapshots

 $t = 293.90 \tau_{A}$

t = 0.0



Pseudocolor Var: p 0.05061 1.0 0.03794 - 0.02526.5-0.01259 0.0 N -8.897e-05 Max: 0.05061 Min: -8.897e-05 -0.5 -1.0 -1.5 0.2 0.4 0.6 0.8 1.0 1.2 1.4 1.6 Major Radius n=1 instability occurs while VDE is stalled.

 $t = 453.54 \tau_{A}$



VDE accelerates following thermal quench.

Poincaré plots



Time History





Halo Current Distribution at Peak



Conclusions

• When plasma resistivity is large compared to wall resistivity ($\eta_0/\eta_{wall} > 10^{-3}$), the *n*=1 instability occurs before the VDE begins. This instability is very energetic, but dissipates heat before the plasma contacts the wall, resulting in relatively low disruption halo currents and forces.

• When plasma resistivity is small compared to wall resistivity ($\eta_0/\eta_{wall} < 10^{-4}$), the VDE is completed before the n=1 mode is destabilized. This results in higher divertor heat flux and halo fraction (since the thermal and current quenches have not occurred before the plasma makes contact), but very low toroidal peaking factor and net sideways force (since the plasma remains essentially axisymmetric throughout the disruption).

• When plasma resistivity is such that the *n*=1 and *n*=0 modes are destabilized on the same time scale, the plasma that hits the wall is non-axisymmetric but still very energetic, resulting in much higher TPF and sideways forces and somewhat elevated halo fraction and vertical force.

• In all cases, the vertical force on the vessel is greater than the net horizontal force by more than an order of magnitude.

Future Work

- The wall time for most of the cases to date is unrealistically small ($\tau_{\rm wall}$ for NSTX is estimated to be roughly 10 ms).
- The case with the most realistic wall time is initialized with spuriously large downward velocity and still seems to undergo a resistive 2,1 instability before getting close to the wall.
- Density evolution can alter the dynamics but has been omitted thus far because of problems with numerical stability.
- Therefore, ongoing work involves running with a more ideal plasma and attempting to stabilize the numerics in the presence of a density gradient.
- It remains to be determined how best to validate the results against NSTX observations.