An Improved Tokamak Sawtooth Benchmark for 3D Nonlinear MHD J.A. Breslau¹, C.R. Sovinec², S.C. Jardin¹ ¹Princeton Plasma Physics Laboratory, P.O. Box 451, Princeton, NJ 08543 ²University of Wisconsin-Madison College of Engineering, 519 Engineering Research Building, 1500 Engineering Drive, Madison, WI 53706-1609

The sawtooth instability [1] is a fundamental dynamic of inductive tokamak discharges. Accurate prediction of the sawtooth cycle is therefore an important test for nonlinear MHD codes. The sawtooth cycle in the CDX-U tokamak [2], chosen because its small size and low temperature allow simulation using actual device parameters, has been an important benchmark for the comparison of the M3D [3] and NIMROD [4] codes for the last several years. Successive comparisons have led to improvements and refinements in both codes, such as a more conservative form of the toroidal current evolution equation in M3D; and a more accurate equilibrium pre-processor in NIMROD. The most recent comparisons show impressive agreement between the two codes both on the linear instability of the m=1, n=1 mode that gives rise to the sawtooth crash, and on the details of nonlinear cyclical sawtooth behavior. Because these tests are somewhat idealized however, they do not vet agree quantitatively with the experimentally observed sawtooth period. We are now standardizing a second generation of CDX sawtooth benchmarks based on an analytically specified equilibrium and a thermal transport model that shows greater fidelity to the physical device. Other improvements include the use of a loop voltage and ohmic heating to replace prescribed source profiles. The specification of this benchmark will be published as a standard test problem, available for testing by other plasma codes.

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1. Introduction

Verification and validation of 3D nonlinear MHD initial value codes is a particularly challenging task. The inherent high sensitivity of nonlinear systems to small differences in initial conditions makes it difficult to distinguish the effects of differences in representation or time-advance scheme from differences in fidelity to the physical model when making detailed comparisons of the predictions of two different codes for a particular instability or other event. It is still more difficult to compare such predictions directly with experimental observations, for which measurement error leads to far greater uncertainty regarding the initial conditions. Nevertheless, such efforts are necessary to justify confidence in the predictive capabilities of these codes.

A program of verification and validation has been undertaken for the two workhorse 3D MHD codes of the SciDAC Center for Extended MHD Modeling (CEMM), M3D [1] and NIMROD [2]. The nonlinear instability chosen for the test is the resistive internal kink mode that gives rise to the sawtooth crash [3], a fundamental dynamic of the inductive tokamak discharges these codes are intended to model. The crash involves magnetic reconnection across a thin helical current sheet, a structure whose size varies roughly inversely with the plasma temperature. For large magnetic fusion experiments, the high temperatures result in a current sheet too thin to be practically resolved by present-day codes. Some small tokamaks, however, such as CDX-U [4], are cold enough to have resolvable current layers and hence make good targets for validation studies using actual device parameters. CDX-U was chosen for this study.

In this article, we will present results of the first successful CEMM cross-code nonlinear verification benchmark, the CDX-U sawtooth cycle. Because we believe that this problem could be of value to the larger fluid modeling community, we also propose a new version of the benchmark with an analytically specified initial state. Preliminary results with this new equilibrium are then presented.

2. Statement of the Problem

CDX-U is a small (R_0 =33.5 cm), low-aspect-ratio (R_0/a =1.5) tokamak experiment with a typical operating temperature of about T_e =100 eV. Modeling 3D macroscopic activity in the experiment requires information on profiles, sources, and transport coefficients. The equilibrium profiles and inferred sources are provided by running the 2D transport timescale code TSC [5] to match typical traces of the plasma current $I_p(t)$ from the experiment. A sequence of experimentally relevant profiles, each at a fixed time, are obtained from the TSC computation as described in [6]. We note that as the central current density increases in the TSC calculation, the central safety factor q_0 , a measure of the pitch of the local magnetic field, falls below unity, the condition for onset of the kink instability. For the equilibrium in the initial benchmark, q_0 =0.92.

M3D and NIMROD are both parallel 3D nonlinear magnetohydrodynamic (MHD) codes in toroidal geometry, solving a superset of the resistive MHD equations that describe the

behavior of a collisional magnetized plasma on timescales long compared to electrostatic oscillations but typically short compared to resistive diffusion:

$$\frac{\partial \rho}{\partial t} + \nabla \cdot (\rho \mathbf{v}) = 0 \tag{1.1}$$

$$\rho\left(\frac{\partial \mathbf{v}}{\partial t} + \mathbf{v} \cdot \nabla \mathbf{v}\right) = \mathbf{J} \times \mathbf{B} - \nabla p + \mu \nabla^2 \mathbf{v}$$
(1.2)

$$\frac{\partial \mathbf{B}}{\partial t} = \nabla \times (\mathbf{v} \times \mathbf{B} - \eta \mathbf{J}) \tag{1.3}$$

$$\mathbf{J} = \nabla \times \mathbf{B} \tag{1.4}$$

$$\frac{\partial p}{\partial t} + \mathbf{v} \cdot \nabla p = -\gamma p \nabla \cdot \mathbf{v} + \nabla \cdot \left[\kappa_{\parallel} \nabla_{\parallel} \left(\frac{p}{\rho} \right) + \kappa_{\perp} \nabla_{\perp} \left(\frac{p}{\rho} \right) \right]$$
(1.5)

where ρ is the number density, v is the bulk velocity, J is the current density, B is the magnetic field, p is the pressure, μ is the viscosity, η is the resistivity, $\gamma=5/3$ is the ratio of specific heats, and κ_{\parallel} and κ_{\perp} are coefficients of heat diffusion perpendicular and parallel to the magnetic field respectively. However, the two codes have a number of fundamental differences in numerics. Both use finite element approaches on 2D poloidal meshes, but M3D uses linear basis functions on an unstructured triangular mesh whereas NIMROD uses high-order finite elements on a structured rectangular mesh. NIMROD represents the toroidal dimension with a spectral representation with each 2D mesh corresponding to a particular toroidal mode number, while M3D uses finite differences toroidally, with each 2D mesh corresponding to a particular value of toroidal angle ϕ . In NIMROD, the linear terms of (1.1)-(1.5) that apply to individual mode numbers and the nonlinear terms that couple different modes together are computed separately, while in M3D these terms are not separated. As a result of these design choices, NIMROD is considerably more efficient than M3D at computing linear modes but potentially less so in the nonlinear regime when the perturbation becomes comparable in size to the equilibrium. NIMROD, however, uses a fully implicit time advance for the linear terms while M3D's scheme is only partially implicit, with a time step limited by the Courant condition on the shear Alfvén wave; as a result, NIMROD typically runs faster on cases like the one described here.

For the initial benchmark, it was agreed that the codes would use a non-evolving Spitzer resistivity profile of the form $\eta \propto T_{eq}^{-3/2}$, normalized so that the central Lundquist number $S=1.94\times10^4$. The viscosity was held constant and uniform with a central Prandtl number $P_N \equiv \mu/\eta$ of 10. A high, uniform, and constant value of κ_{\perp} was chosen, equivalent to a physical value of approximately 200 m²/s. The two codes differ in their implementation of parallel heat diffusion, with M3D using an "artificial sound wave" method in which a hyperbolic rather than parabolic operator convects heat along the field while NIMROD evaluates the actual diffusion term in (1.5). The parallel heat conduction parameter could therefore only be approximately similar in the two runs; it was chosen to correspond to an electron thermal speed six times the Alfvén speed. Source terms were added in the field

(1.3) and energy (1.5) equations to drive these respective profiles toward their initial values.

3. Initial Study

The first comparison made was of the linear n=1 eigenmode structure and growth rates. As expected based on earlier successful linear benchmarks, agreement here was good (Fig. 1). Both codes found the most unstable n=1 mode to have dominant poloidal mode number m=1 (the expected kink mode at the q=1 surface). M3D predicted a normalized growth rate of $\gamma \tau_A = 5.1 \times 10^{-3}$, and NIMROD $\gamma \tau_A = 4.4 \times 10^{-3}$.



Fig. 1: Filled contour plots of toroidal current density on a constant- ϕ section for the *n*=1 eigenmode of the q_0 =0.92 CDX-U equilibrium. a. M3D result. b. NIMROD result.

The initial nonlinear comparison was less successful. A previously published set of results from M3D reproduces many of the expected features of the sawtooth cycle [6] but shows disagreement with the NIMROD prediction (Fig. 2).



Fig. 2. Time history of kinetic energy by toroidal mode number during the first two sawtooth crashes in an earlier iteration of the nonlinear CDX benchmark. a. M3D result (normalized units). b. NIMROD result (mks units).

While the observation of Kadomtsev reconnection [7], temperature flattening, and repeating cycles was promising, this attempt had to be regarded as a failure with respect to code verification. As illustrated in the figures, the two predictions did not agree on the degree of damping of oscillation strength between crashes, on the sawtooth period (480 Alfven times for M3D; 800 for NIMROD), or on the growth rate of the n=1 mode in the linear phase, which remains constant in NIMROD but increases in time significantly in M3D. There were also indications that the M3D case was under-resolved and not convergent toroidally.

The changing growth rate effect in M3D was found to arise from an unphysical steady decrease in the central value of the safety factor q_0 arising from poor conservation properties of M3D's treatment of toroidal current density in the presence of a current source term and equilibrium flow. This poor conservation was shown to result in a significant increase in the central current density over time (and hence a decrease in q_0) even in a two-dimensional nonlinear simulation with no n=1 trigger. It arose numerically from a scheme in which the Laplacian of the expression for the rate of change of the poloidal flux function $\psi(\mathbf{x}, t)$ was evaluated algebraically and then assembled term by

term to compute the rate of change of the current density $C_a = \Delta^* \psi \equiv R^2 \nabla \cdot \left(\frac{1}{R^2} \nabla \psi\right).$

Following the first M3D-NIMROD comparison, it was realized that much more accurate conservation could be achieved by composing $\dot{\psi}$ term by term instead and then evaluating the Laplacian numerically. Figure 3 illustrates the improvement in conservation accomplished by this change.



Figure 3: Profile of toroidal current density along midplane at several times during nonlinear 2D M3D run beginning with unperturbed CDX equilibrium. a. Original C equation. b. Improved, conservative C equation.

Other improvements that were found to be required included the substitution of an isotropic viscosity operator in M3D for the perpendicular operator that was originally in use; and a more accurate equilibrium calculation in NIMROD in which the parallel component of the current density was recomputed from the equilibrium magnetic field on initialization rather than interpolated from the input file using cubic splines. Precise specifications of the transport coefficients were also reconsidered carefully to ensure that

differences in normalization conventions in the respective codes did not result in each solving a different problem (apart from unavoidable differences in the respective models, e.g. for parallel heat transport). Finally, a scaling study in q_0 revealed that the n=1 growth rate was highly sensitive to this parameter and that the $q_0=0.92$ equilibrium was close to marginal stability with the transport coefficients listed earlier, making detailed agreement especially challenging. Accordingly, a different equilibrium extracted from the same TSC sequence, with $q_0=0.82$, was chosen for the follow-up study when it was found that the codes were in robust, converged agreement on its n=1 eigenmode and growth rate.

5. Agreement

The results obtained after the aforementioned fixes are shown for comparison in Fig. 4.



Figure 4: Time history of kinetic energy by toroidal mode number during the first three sawtooth crashes in the present iteration of the nonlinear CDX benchmark. a. M3D result (energy in normalized units, time in μ s). b. NIMROD result (mks units).

It is clear from the figure that the codes are now in substantial, detailed agreement. Like NIMROD, M3D now conserves q_0 in the absence of mode activity, and thus shows a constant linear growth rate for the n=1 mode until just before the first crash. The codes also agree in the relative magnitudes of the various toroidal modes before, during, and

after the crashes; on the detailed time behavior of the low-n modes; on the degree of damping of the oscillation in successive cycles, and on the cycle period of $\sim 200 \ \mu s$. It has also been confirmed that the M3D result is now converged toroidally.

When we investigate the actual plasma state at various corresponding times in the two runs, we also find detailed agreement (Figs. 5-6).



Figure 5: Poincaré sections showing magnetic flux surfaces at several time instants during the M3D (top) and NIMROD (bottom) sawtooth cycles. Left: late linear phase. Center: at culmination of crash. Right: Early recovery phase following crash.



Figure 6: Temperature contours at two times during the M3D (top) and NIMROD (bottom) sawtooth cycles. Left: at culmination of crash. Right: Early recovery phase following crash.

This agreement constitutes a successful verification of the two codes. However the modeling is insufficient to produce quantitative agreement with the experimental results for the two predictions that can be compared directly with soft X-ray data from the experiment: the sawtooth period and the crash time . The predicted period of 200 μ s is significantly less than the observed 500 μ s sawtooth period in CDX, and the predicted crash time is a much larger fraction of the total cycle time than is observed in the device. Hence this study cannot be considered a successful validation of the model. A more refined model and a set of initial and boundary conditions that show greater fidelity to the experimental conditions are required.

6. New proposed Equilibrium, preliminary results

Two sets of considerations governed the formulation of the next iteration of the CDX sawtooth benchmark. The first was to match the physics of the experiment more closely in terms of boundary conditions, sources, and transport coefficients so that these differences do not interfere with a successful validation. The second was to define the new initial equilibrium state in a simple analytic way so that the conditions would be absolutely clear to both code groups. This would also enable any other nonlinear predictive tokamak MHD code to be tested on the same problem, making possible a standard community-wide benchmark.

CDX is a small, inductively-driven short-pulse tokamak without any auxiliary heating or current drive sources. In contrast, the heat and current sources present in the previous test always drive these profiles toward their initial equilibrium states, which is unphysical. For this iteration, these sources are removed. Instead all heating is now self-consistent ohmic heating, with a resistivity profile that evolves to track the temperature with a $T^{-3/2}$ dependence, rather than remaining static as before. The very high, uniform perpendicular heat conduction is replaced by a static profile that more accurately represents what was inferred from the experiment. And current drive is now purely inductive, with a loop voltage applied as a boundary condition and regulated to provide constant power input during the run.

The new equilibrium is specified analytically in table 1. The equilibrium may be computed with any equilibrium code, such as JSOLVER [8]. The initial, uniform loop voltage is that needed to drive the initial toroidal current against the initial resistance:

$$V_{loop} = \frac{2\pi\eta \langle \mathbf{J} \cdot \mathbf{B} \rangle}{\langle \mathbf{B} \cdot \nabla \phi \rangle}$$
(1.6)

where the brackets denote the flux surface average. Subsequently, the voltage should track the evolving current to maintain constant input power. The perpendicular heat conduction profile should be computed self-consistently to provide a steady state in the presence of the ohmic heating arising from the current density and temperature distributions – if the random heat flux is $\mathbf{q} = -\kappa_{\perp} \nabla_{\perp} T$, then we can integrate the surface average of the energy equation to get

$$\kappa_{\perp} = \frac{1}{T' \left\langle \left| \nabla \psi \right|^2 \right\rangle} \left[\frac{V_{loop}}{2\pi\mu_0} \left\langle \frac{\left| \nabla \psi \right|^2}{R^2} \right\rangle \right]$$
(1.7)

where the prime denotes the derivative with respect to the poloidal flux function ψ and the particle flux corresponding to Pfirsch-Schlüter diffusion has been neglected.

Quantity	Value
Major radius R ₀	0.341 m
Minor radius <i>a</i>	0.247 m (aspect ratio = 1.38)
Ellipticity ĸ	1.35
Triangularity δ	0.25
Central temperature $(T_e=T_i)$	100 eV
Normalized central pressure $\mu_0 p_0$	2.5×10^{-4} (implies $n_0 = 1.8 \times 10^{-19} \text{ m}^{-3}$)
α Parameter in pressure equation*	0.1
Vacuum value g_0 of $R \cdot B_T$	0.042 T·m
Effective ion charge Z _{EFF}	2.0
Loop voltage $V_{\rm L}$	3.1741 V (implies $q_0 \approx 0.82$)
* $n(\psi) = n \left[\alpha \tilde{\psi} + (1 - \alpha) \tilde{\psi}^2 \right]$ where $\tilde{\psi} = \frac{\psi - \psi_{\text{limiter}}}{\psi - \psi_{\text{limiter}}}$	

Table1: Parameters for the next equilibrium for the CDX sawtooth benchmark.

* $p(\psi) = p_0 \left[\alpha \tilde{\psi} + (1 - \alpha) \tilde{\psi}^2 \right]$, where $\tilde{\psi} = \frac{\psi - \psi_{\text{limiter}}}{\psi_{\text{axis}} - \psi_{\text{limiter}}}$.

The plasma boundary is parameterized as follows based on the values in the table:

$$R(\theta) = R_0 + a\cos[\theta + \delta\sin(\theta)]$$

$$z(\theta) = a\kappa\sin(\theta)$$
(1.8)

The temperature profile is taken to be linear in the normalized flux,

$$T(\boldsymbol{\psi}) = T_0 \tilde{\boldsymbol{\psi}},\tag{1.9}$$

so that the density becomes

$$n(\psi) = \frac{p}{2k_B T} = \frac{p_0}{2k_B T_0} \left[\alpha + (1 - \alpha)\tilde{\psi} \right]$$
(1.10)

where $k_B = 1.6022 \times 10^{-19}$ J/eV is Boltzmann's constant. The Spitzer resistivity profile has a coefficient of $1.03 \times 10^{-3} Z_{EFF} \Omega$ -m, which assumes the Coulomb logarithm ln Λ =20.

Initial axisymmetric nonlinear M3D results with this new equilibrium show promise, confirming that the steady state can be maintained without conventional sources.

Preliminary low-n linear results now suggest that this equilibrium is susceptible to the same 1,1 mode as the previous one, but with a somewhat higher growth rate; and that its higher-n modes are unstable at the same rational surface, with smaller growth rates.

7. Conclusions and Future plans

Our original study demonstrated that detailed nonlinear MHD simulation of the sawtooth cycle with representative device parameters was possible for a small device and with sufficient computing power, and that it was capable of reproducing qualitatively all the salient features of the instability. We have now shown that cross-code verification is possible in the nonlinear regime as well, with close quantitative agreement achieved in spite of the numerous differences in numerical implementation. This exercise, while time-consuming, was extremely valuable for both groups of developers in that it led to greater understanding (and in many cases improvements) of each code's behavior, and ultimately to enhanced confidence in the accuracy of their predictions. It also provided a lesson in the necessity of thorough, clear communication in the preparation of such a complex nonlinear benchmark.

Based on the lessons learned in the successful verification exercise, and out of a desire for a successful validation as well, we have proposed a new more rigorous and more physically valid test based on the same instability. We have constructed the new equilibrium and shown that the conditions of the new test lead to the same instability. We intend to pursue this instability into the nonlinear regime once again with both codes and hope to achieve a three-way agreement between them and the experiment. We also invite other members of the tokamak modeling community to adopt this test problem as a nonlinear benchmark and to share their results.

The goal of this research is ultimately to develop a predictive capability relevant to burning plasma fusion experiments that are much larger and much hotter than CDX-U. While this verification result is gratifying, simple scalings indicate that many orders of magnitude more computing power would be needed to apply these codes to those experiments in a straightforward way using their real physical parameters. While some of this increase may come from future hardware, much of it must be achieved by advances in numerical methods, such as the use of greater implicitness, adaptive mesh refinement, and more efficient linear solvers.

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