

# Disruption Impact Reduction

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# Brief comment on FSP Goals

- Most difficult part of FSP is in simultaneously satisfying the requirements of:
  1. Being something new:  
different from the base/SciDAC program
  2. Being something important:  
address critical issues to ITER
  3. Being relevant to exascale computing
- Satisfying 1 & 2 has generally meant that if it involves integration with different sub-communities it is FSP, if not it is out
  - It is a process that is ongoing
  - Patience is required

# Multiple causes of disruptions exist

- Long-wavelength instabilities
  - Ideal MHD modes
  - Resistive Wall Modes (RWM)
  - VDEs (n=0 RWM)
  - Tearing modes (but usually when locked)
  - ...
- Transport
  - Hardware/software faults Most common
  - Density limits
  - ...
- External events
  - Pellet injections
  - Shutdown scenarios
  - Impurity gas jets
    - AKA, Mitigated disruptions
  - ...

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Causes

MHD  
instabilities

# Preventing/mitigating disruptions requires a broad approach

- Two needs of modeling:
  - Avoid disruptions entirely
    - Use transport modeling to optimize plasmas that avoid MHD stability or density limits
    - Modeling of feedback stabilization of MHD modes
  - Use modeling to understand what happens during a disruption, and:
    - Understand which operational regimes are most dangerous
    - Provide insight for and quantitative evaluation of mitigation techniques
    - Understand machine designs that can withstand disruptions

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Whole device  
modeling  
Validation  
Kinetic/MHD coupling



Edge physics, ...

# FSP Disruption Science Driver

- How well can we predict the onset of a disruption and what strategies are available to avoid their development?
- What are the effects of runaway electrons and what is the impact of operating regimes on their generation?
- How does impurity transport affect disruption dynamics, and how do we use this information to mitigate the effects?
- What is the impact of disruptions on the material wall, and how can we better design the first wall to handle the thermal loads?
- What are the forces on the vacuum vessel and support forces during a disruption, and how do we improve their design?
- How can we better design disruption mitigation systems?
- What are the best plasma models for simulating plasma disruptions?

# Issues have been raised about MHD boundary conditions during disruptions

## **Issues:**

- How do model the “vacuum” region?
- Do we allow currents through the wall?
- Do we have a plasma inflows/outflows from the wall?



# External kink: demonstration of “vacuum” simulations

- The ‘vacuum bubble’ paper [Rosenbluth, Monticello, Strauss, and White, PF 19, 1987 (1976)] considers external kink with 2D incompressible MHD in long cylinders.
  - Lagrangian methods are used to track plasma-vacuum interfaces (uniform- $J$  cases) and flux surfaces (parabolic- $J$ ).
  - The violent cases show plasma swallowing bubbles.
  - A drag term is used for finding minimal energy states.

Minimal energy states presented in the RMSW reference with  $q(a)$  varied and uniform- $J$  (left) and parabolic- $J$  (right).

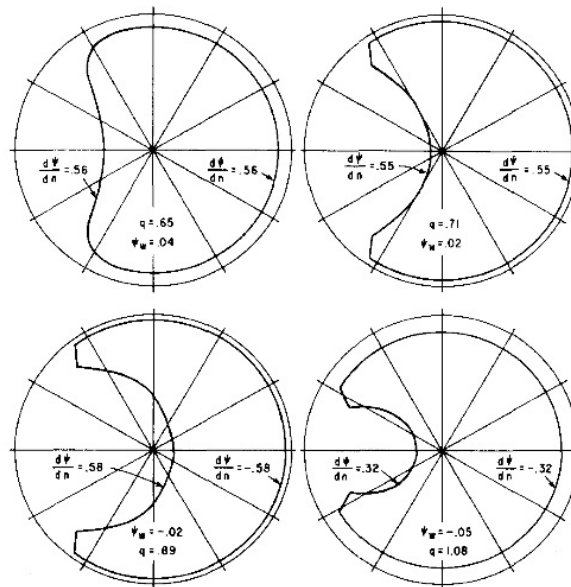


FIG. 5. Minimal energy states,  $\alpha = 0.8$ . Note that  $\partial\psi/\partial n$  is discontinuous for  $\psi_w < 0$ .

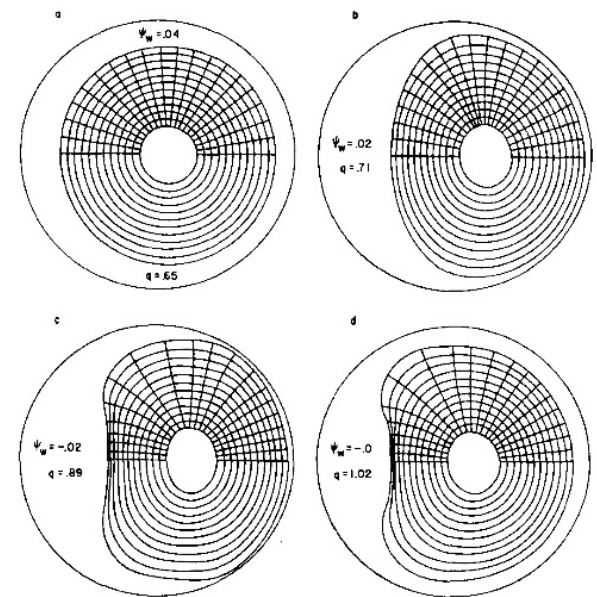
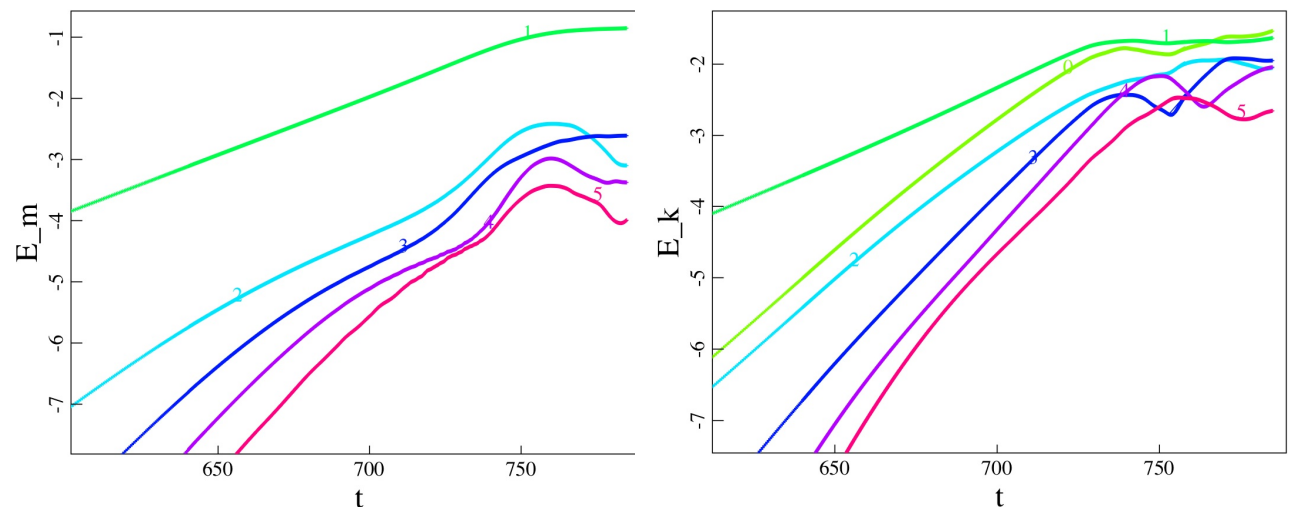


FIG. 10. Minimal energy states for  $m=1$  in the case of a parabolic current profile. Compare with Fig. 5.

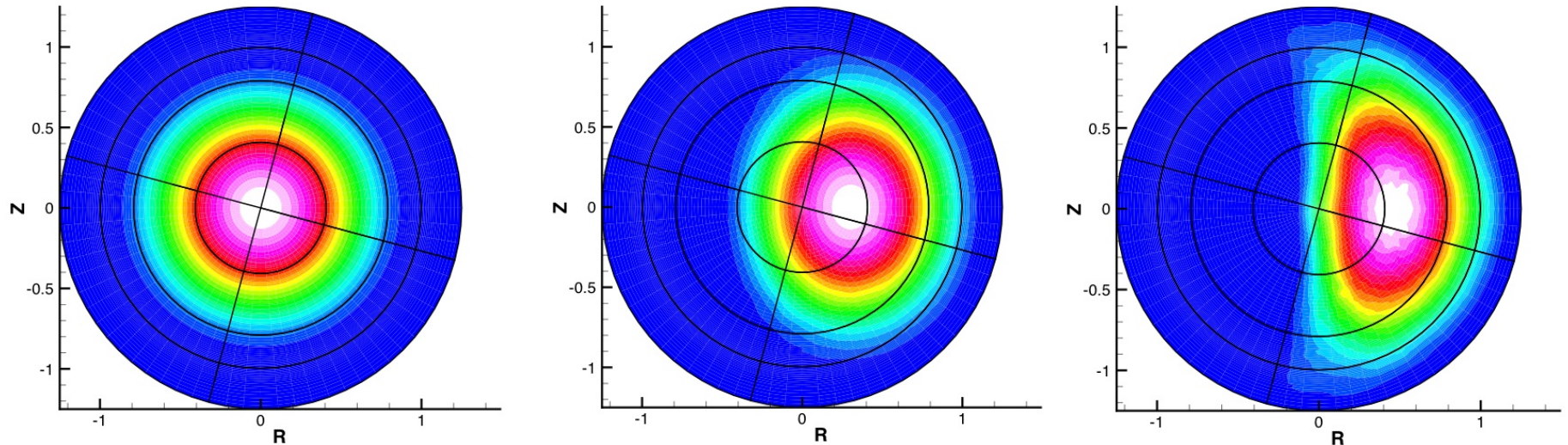
## Minor modifications were made to apply NIMROD to the parabolic- $J$ , $q(a)=0.89$ case.

- For preprocessing, the equilibrium requires no numerical integration ( $B_z$  is uniform), and a hyperbolic-tangent profile for  $n$  is specified.
- In NIMROD, the modifications make  $\eta$  a 3D function of  $n$ ,  $\eta(n) \sim n^{-n_\eta}$ , and  $n_\eta=4$  is used in the nonlinear computations.
- The cylindrical code was used to check that  $B_z/B_\theta(a)=10$  with  $L_z=141a$  produces a mode that is close to the incompressible limit ( $\gamma$  is within 2%).
- A series of linear NIMROD computations check sensitivities to  $n_\eta$ , background density, the width of the transition, and resolution.
- The nonlinear computation presented here has  $\eta(0)=4 \times 10^{-5}$ ,  $\eta_{\text{edge}}=6.4$ ,  $n_{\text{edge}}=0.05$ , some hyper-dissipation, upwinding dissipation, and toroidal components  $0 \leq n \leq 5$ .

**Logarithm of perturbed magnetic (left) and kinetic (right) energy from the nonlinear computation.**



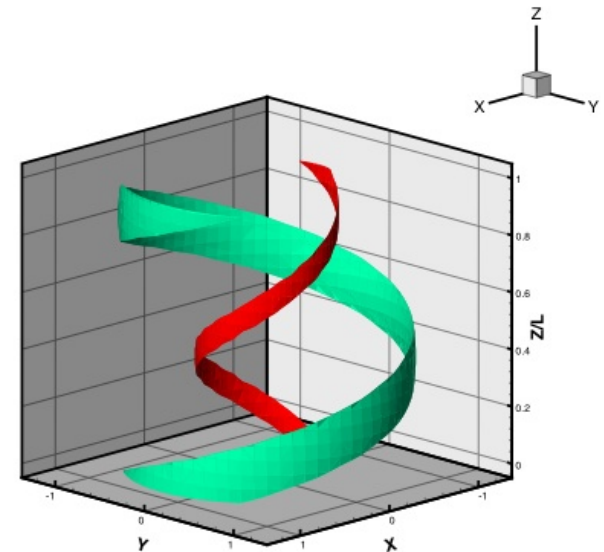
The drag term has not been used in the NIMROD computation; the plasma column distorts into the wall.



Contours of constant pressure at  $t=0$ ,  $3 \tau_A$  from maximum displacement, and  $1 \tau_A$  from maximum displacement.

- Computationally, this case ‘exercises’ NIMROD’s ability to advect sharp fronts in  $n$  and to solve linear systems with very strong variation in coefficients over the periodic coordinate.

**Helical surface**  
currents flow parallel to  $J_0$  along the phase of the column that moves inward and anti-parallel along the phase that moves outward



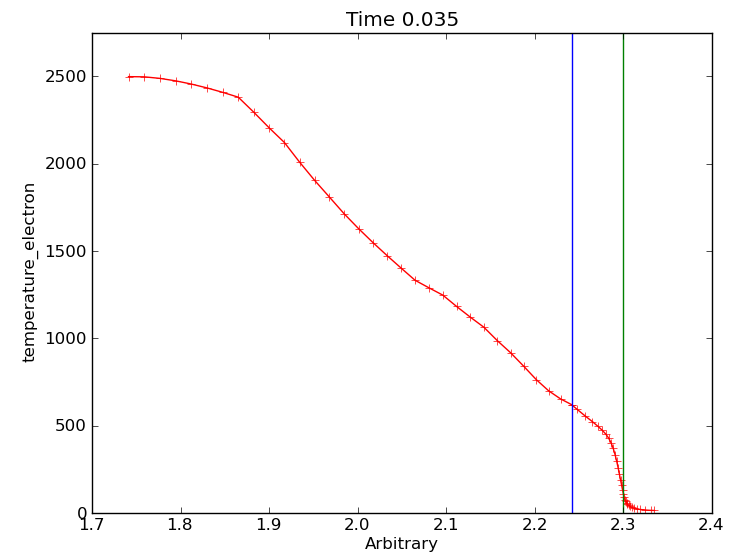
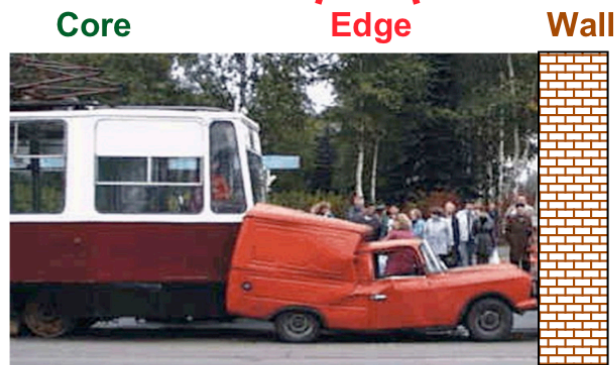
# Modeling of edge plasmas does not use true vacuum region

## “Soft” interface to core

- Continuous physics
- Overlap region, no fixed location
- Couple at a radial point; plasma uniform poloidally
- Likely change of dimension

## “Hard” interface to wall

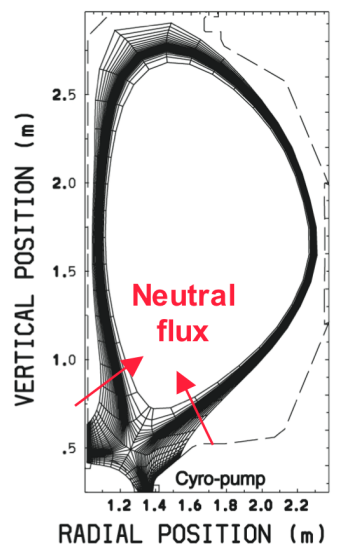
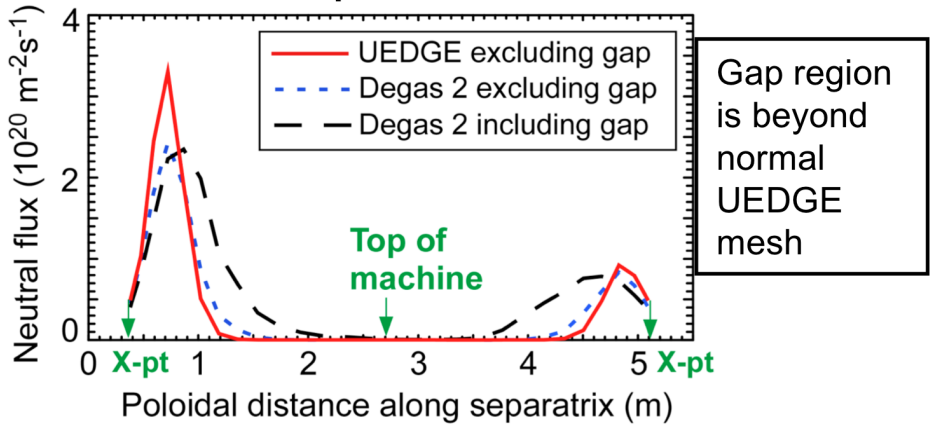
- Discontinuous physics
- Fixed coupling location
- Couple along 1D boundary
- Likely change of dimension



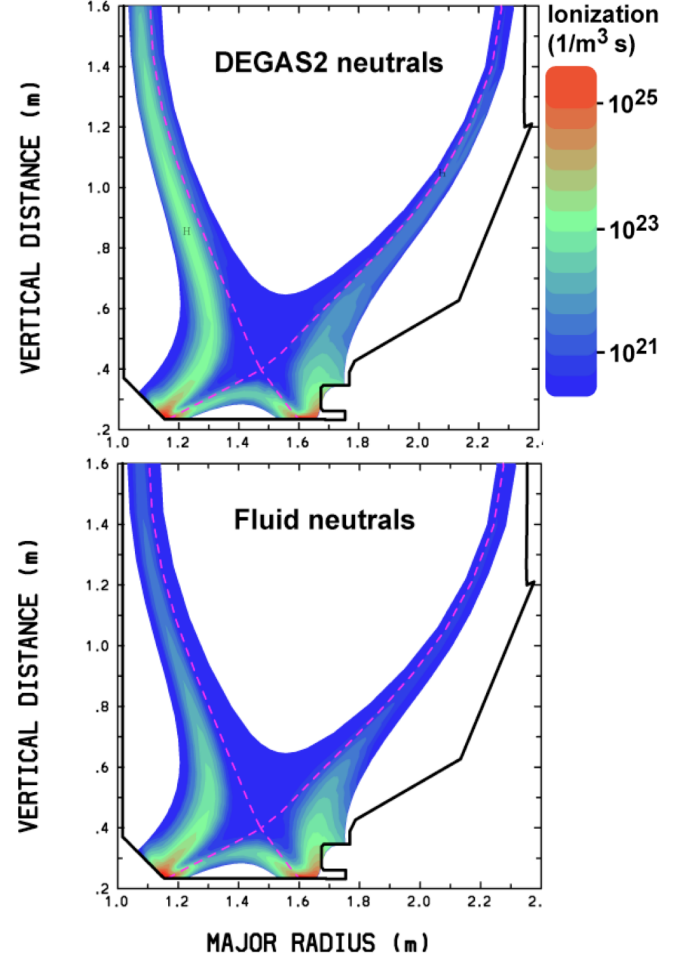


# Edge modeling has traditionally focused on steady states

Radial neutral fueling-flux across separatrix in DIII-D



ELMy H-mode, higher density



Traditional focus: divertor widths, wall heat flux, neutral fueling

# Fluid modeling of steady-state plasmas give information on key physics

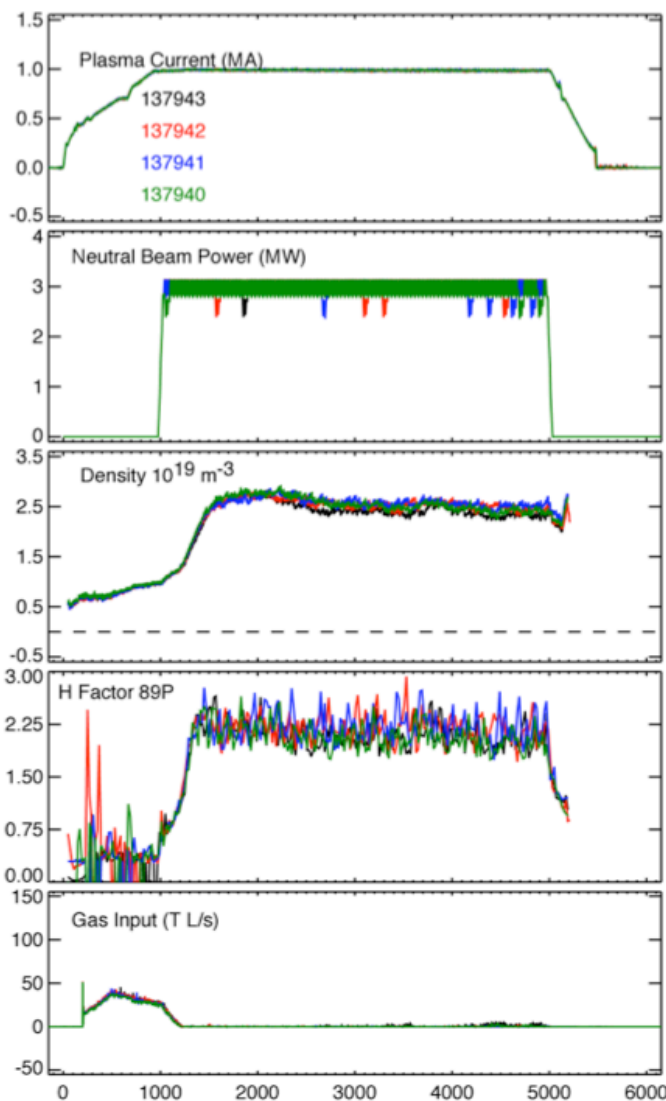
## **Boundary conditions:**

- Sheath boundary conditions  
See: Fundamenski, PPCF 2005 (Review article)
- Recycling coefficients (as input parameter) for neutrals
- Work is ongoing for improving current status by coupling to wall.

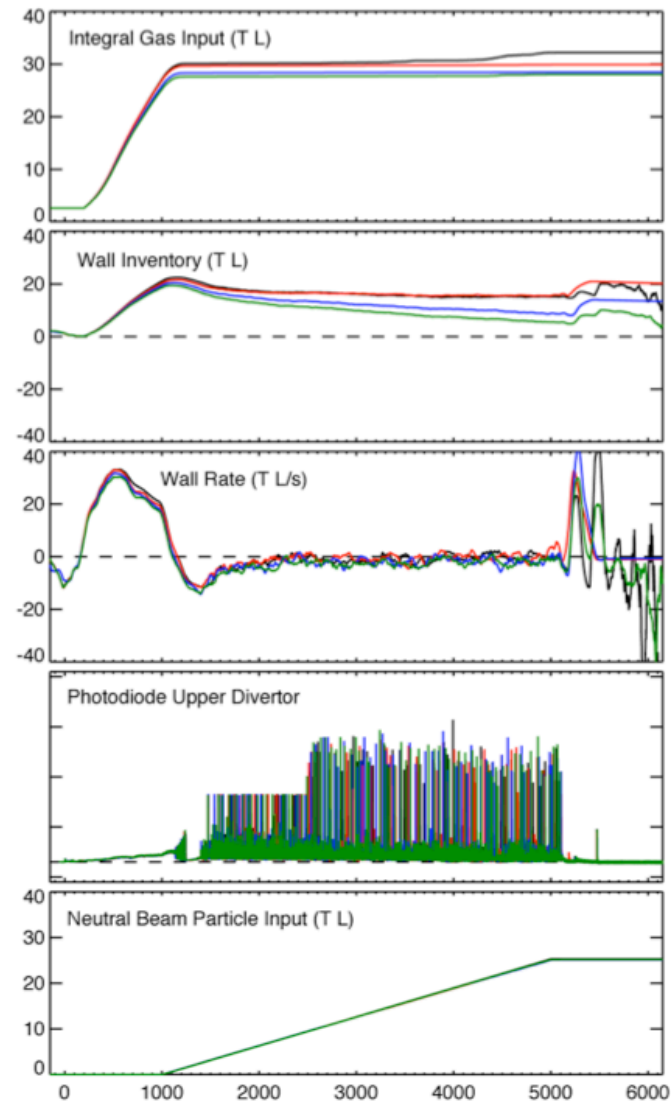
As CEMM codes include more physics, do longer time scale simulations, including this physics increases in importance.

# Change in balance of wall retention occurs during startup/shutdown

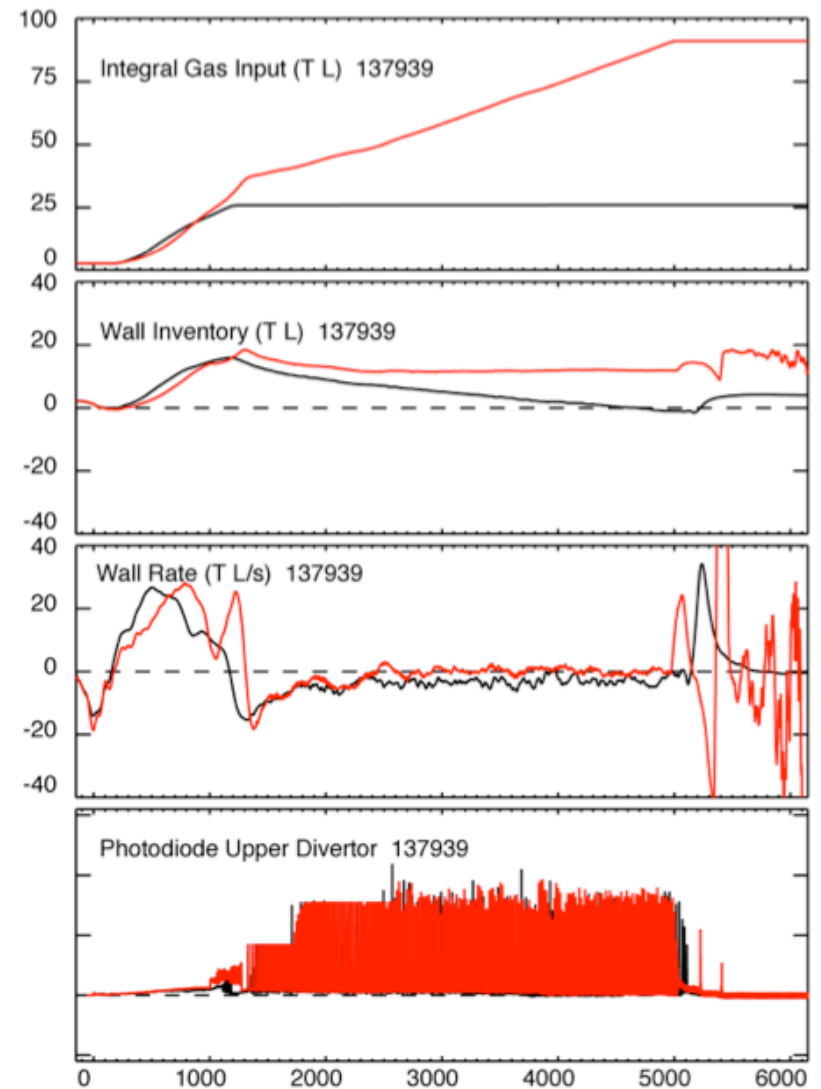
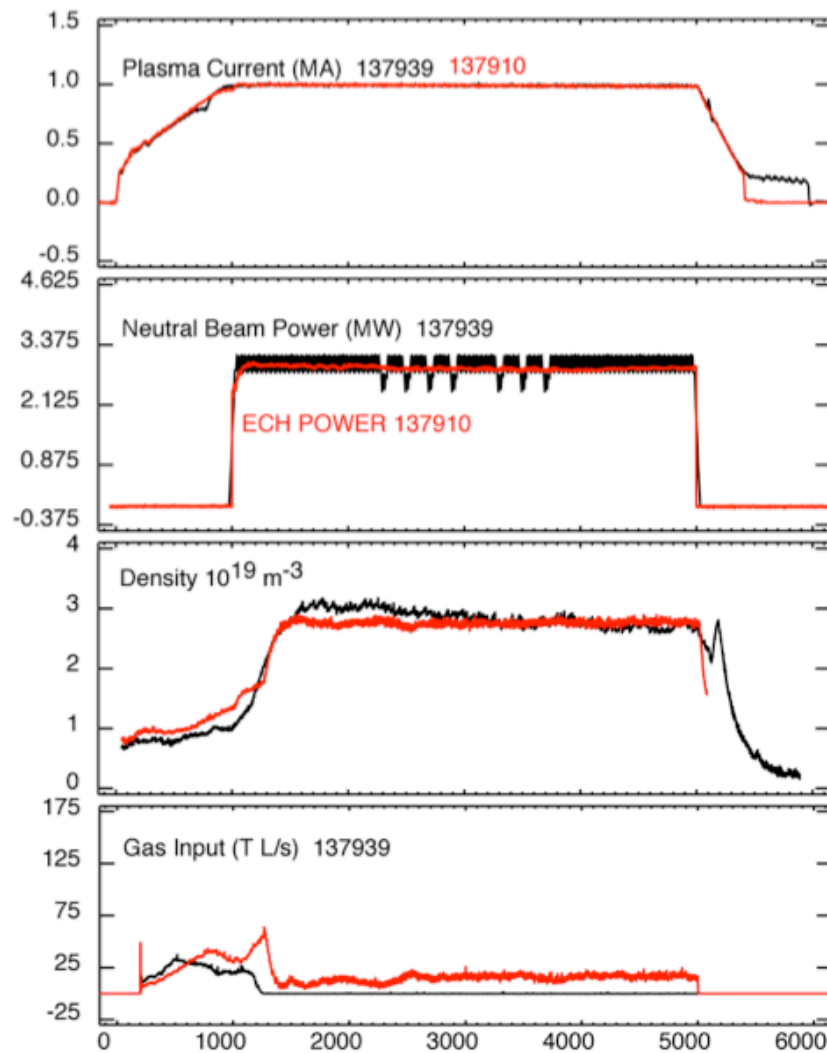
C-MOD:  
Moly Walls



2009 FS



# DIII-D sees similar behavior despite different materials





# Disruptions affect particle balance dramatically

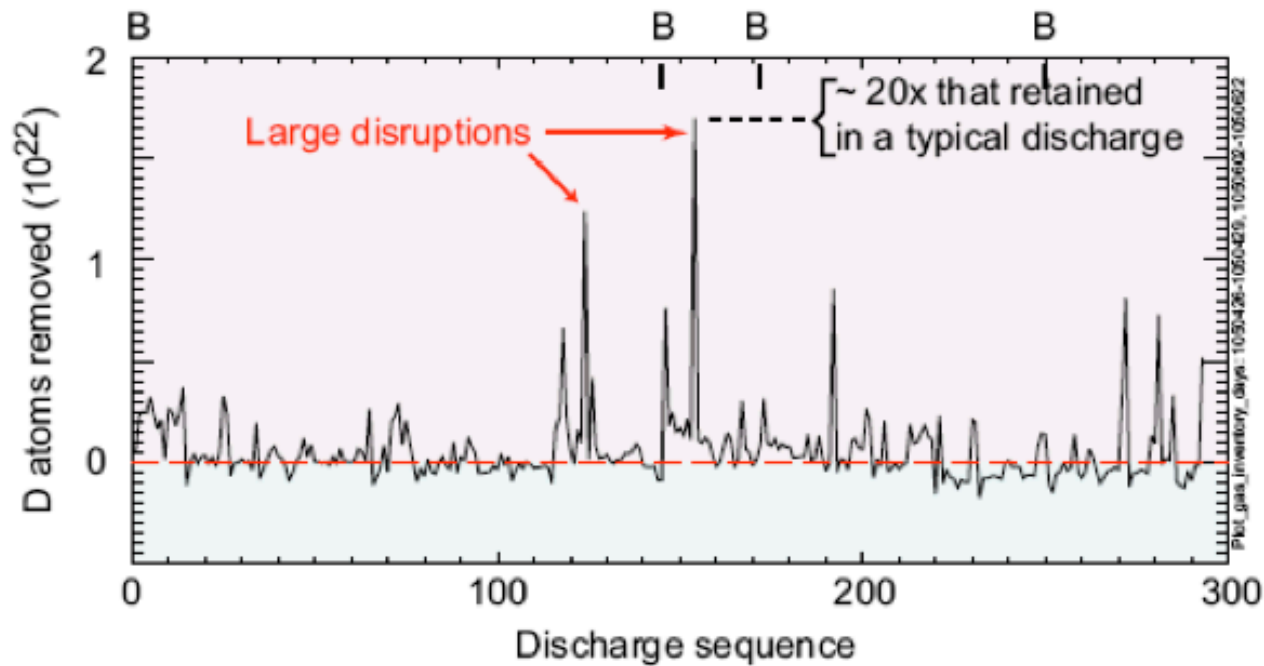
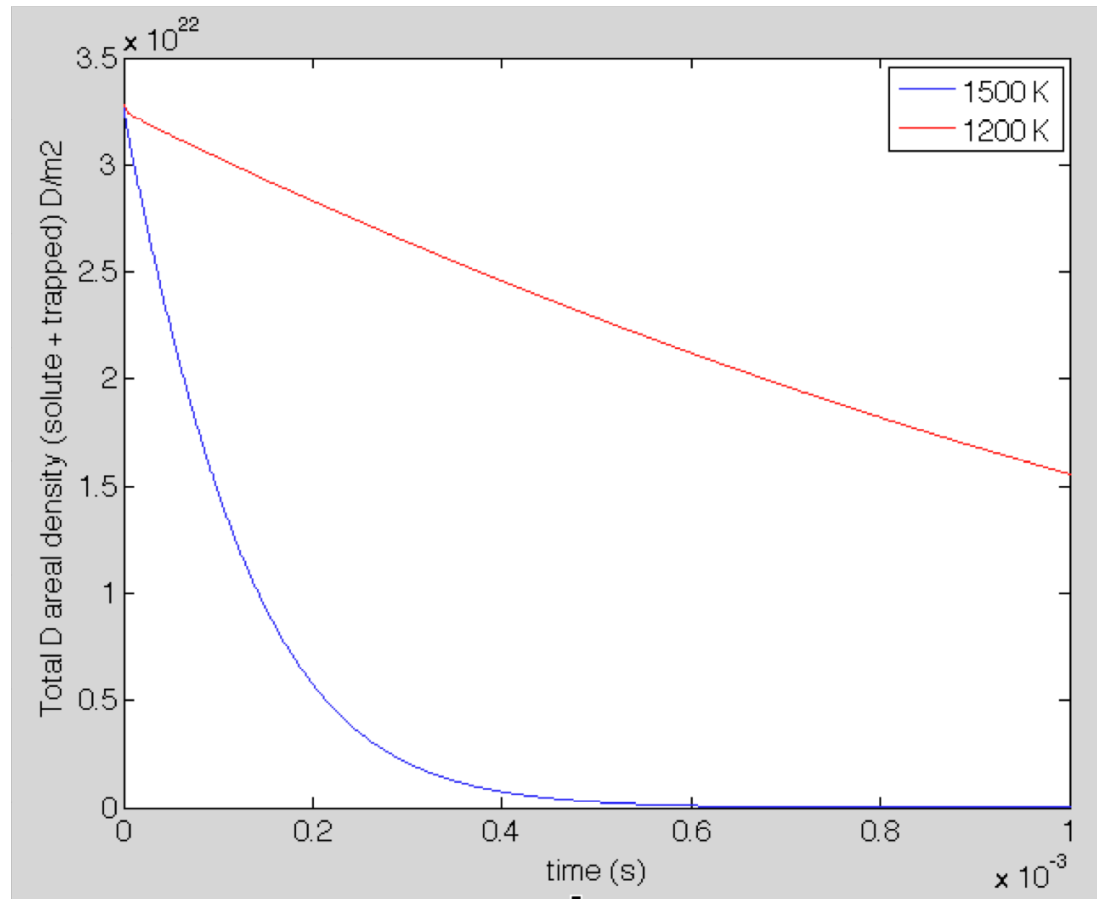


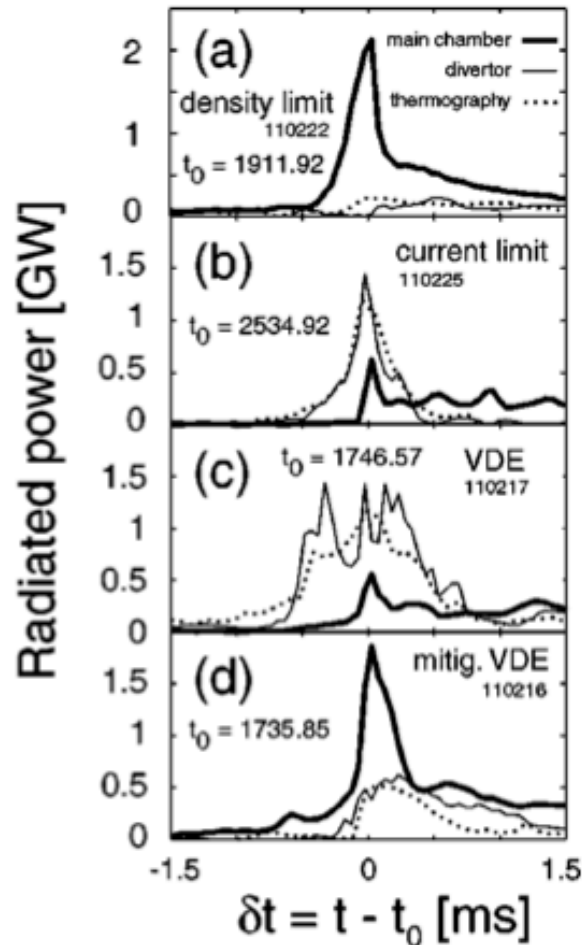
Fig. 11 Shot-to-shot particle balance in C-Mod. B marks times of boronizations [1].

# Modeling of wall retention due to disruption models temperature



*Fig. 21 Numerical simulation of disruption-heating induced reduction of retained D in Mo for two different assumptions of surface temperature.*

# Hydrogenic and impurity inflows can affect disruptions



Hollmann 2003:

- “Self-mitigated” density limit disruption
- Dominant energy loss is radiation losses to main chamber
- Radiation is primarily from impurity generation and neutral outgassing
- Believed to play some role in all disruptions

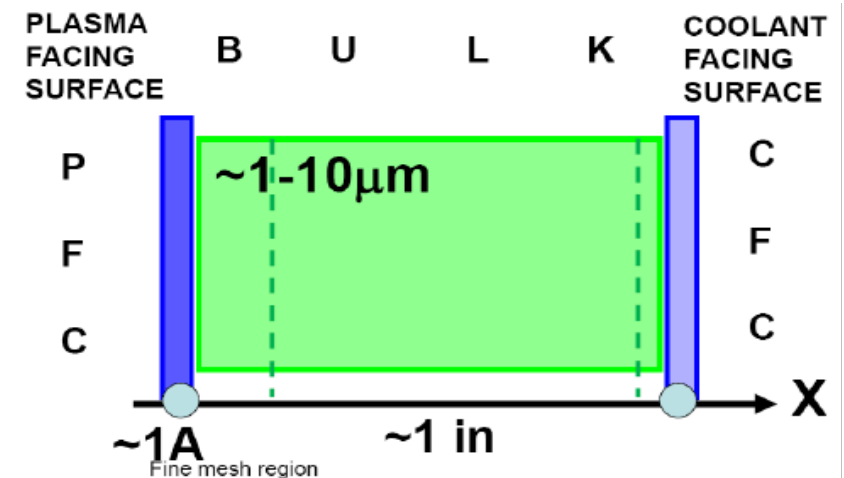
FIG. 3. XUV power radiated from the main chamber and divertor as a function of time for different types of disruptions. Also shown for comparison is divertor thermography data. Time  $t_0$  corresponds to the peak in total radiated power.

# WallPSI is a continuum code to simulate time-dependent 1-D transport of plasma particles and heat in the wall

- Modeling of wall segment includes:
  - 1-D bulk and 0-D plasma facing surface (PFS) & coolant facing surface (CFS)
  - Mobile, adsorbed and trapped hydrogen modeling
  - Wall segment covered with non-uniform mesh w/  $1\text{\AA}$   $\Delta x$  near surfaces
  - Incorporates collisional and thermally activated reactions
  - Diffusive and non-diffusive particle species transport
  - Modeling of reflection coefficients, sputtering yields, hydrogen implantation profile, profile of recoils, and profile of penetrating flux

- **Multiscale**

- **shortest time  $<1\text{ns}$  (treated as collisions)**
- **largest time  $>1\text{year}$  (bulk diffusion, permeation)**



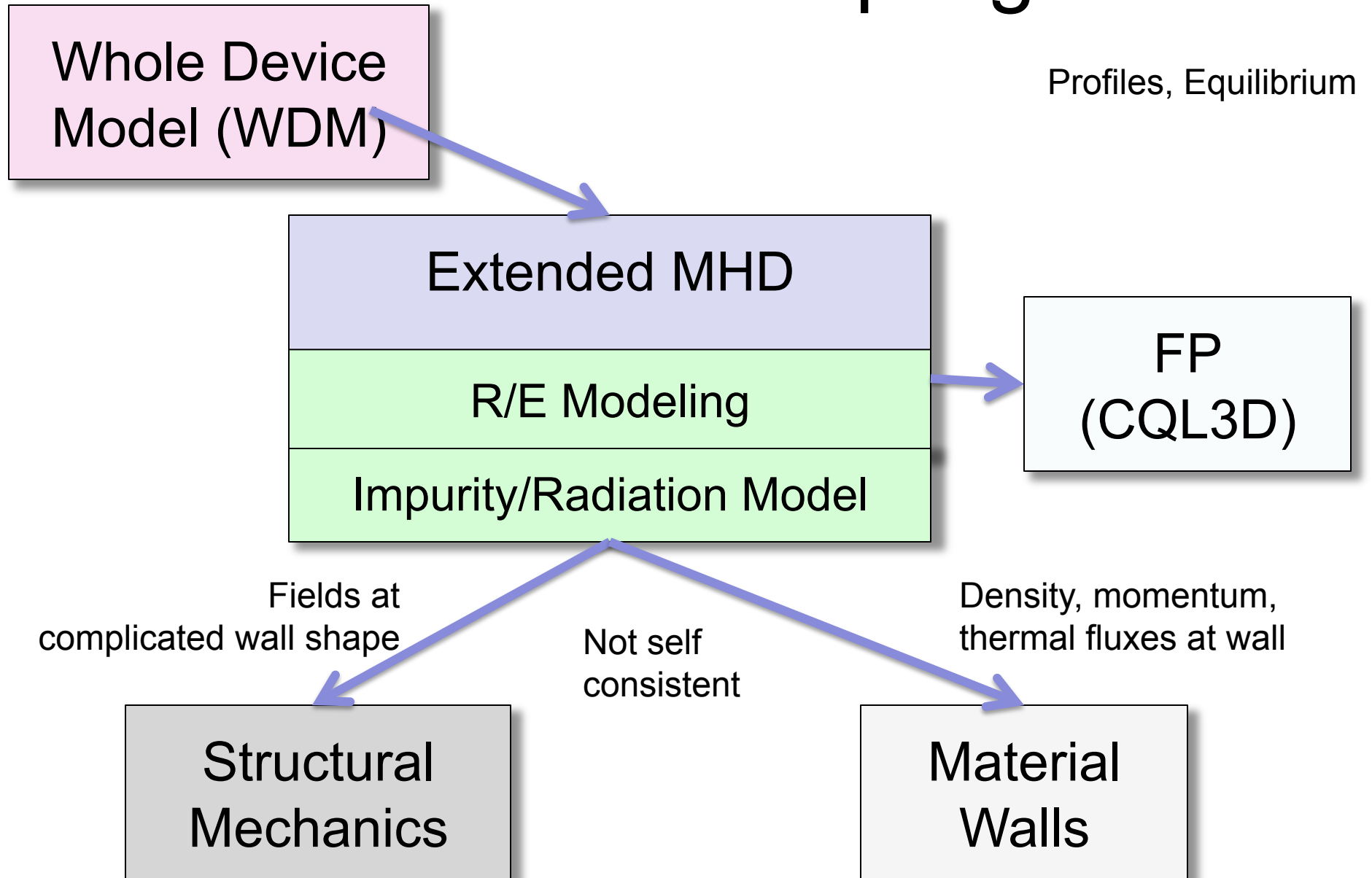
More detailed models can be found in HEIGHTS package

# Implications of edge modeling on CEMM codes

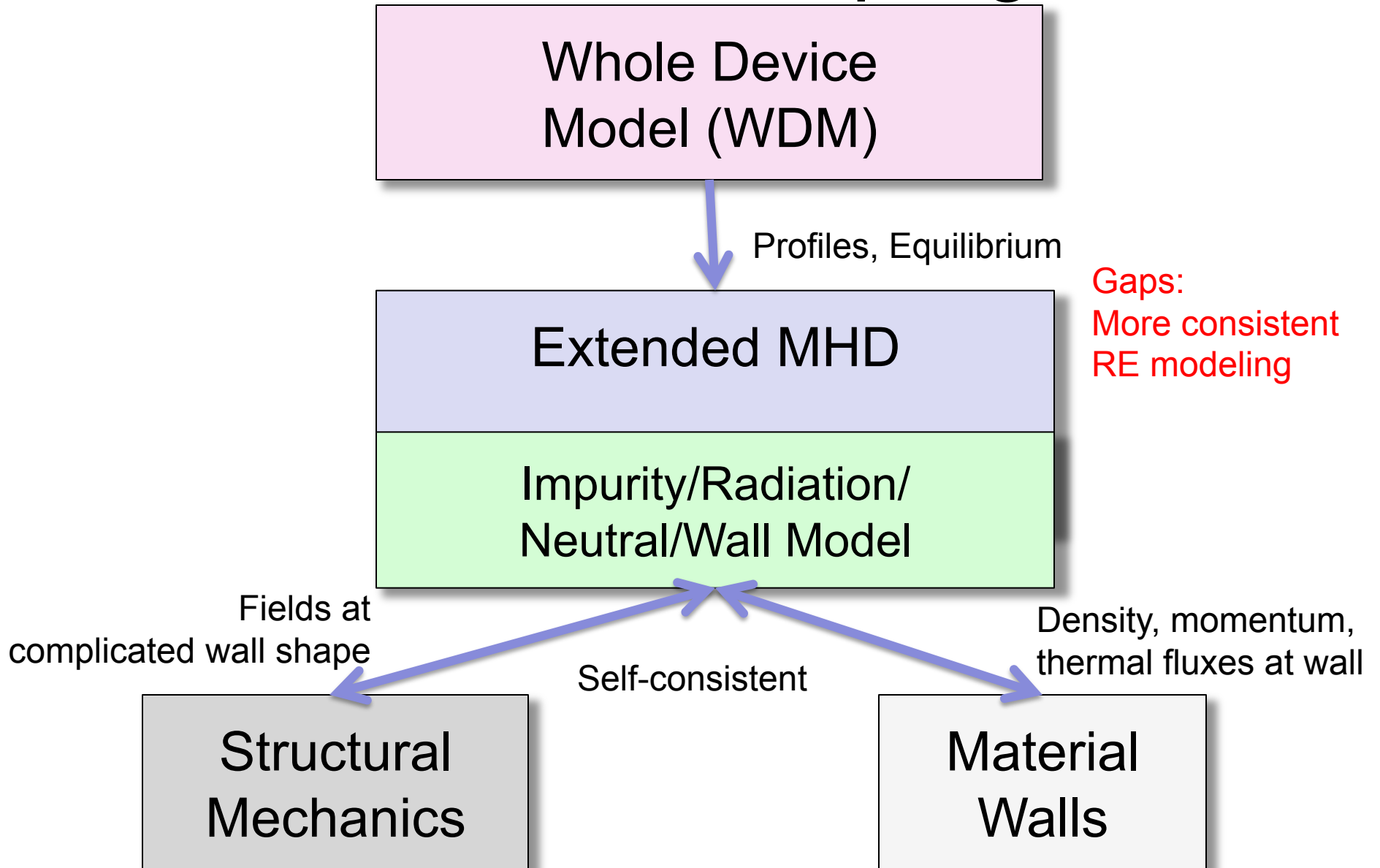
- Near-steady-state conditions are reasonably well-described by sheath boundary conditions, recycling coefficients
- Non-steady-state conditions (ELMs, disruptions) is more difficult
  - No easy model exists – physics is inherently complicated
  - WallPSI offers a reasonable model that might be usable in a nearer time scale
  - HEIGHTS package has many detailed models, but significant caveats exist for this as well
  - Coupling to these packages offers not only the ability to improve the modeling of MHD behavior, but also the ability to study wall retention

Adding this physics is a natural FSP goal

# Near term coupling



# Mid-term coupling



# Comparison with Zakharov's Proposal

Concrete actions proposed:

- Use kinetic models for energy loss
  - Ramos-Held DKE will allow 2<sup>nd</sup> –order accurate electron losses
  - Ions are more problematic – longer term project for discussion
- Use kinetic electrons for RE modeling
  - Started by Izzo. More work needed as indicated in roadmap
- Coupling of codes to gas injections modeling
  - Begun by Izzo. More work needed as indicated in roadmap
- Plasma edge and plasma-wall interactions
- Coupling to vessel conducting structures
- Theory developments
  - Adaptive grid
  - Development of kinetic-MHD hybrids
  - Models for plasma/wall interactions
- Validation