## The effect of lithium conditioning approaches for plasma-facing surfaces on the edge and core temperature and density profiles

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- Introduction
  - LTX and LTX-β
- Progress in the elimination of core plasma temperature gradients - solid lithium walls
- Core impurity levels during tokamak operation with full solid or liquid lithium walls
- Implications of a low collisionality scrape-off layer
- Initial assessment of confinement with  $\nabla T = 0$
- Summary

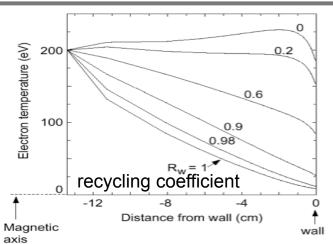


- ◆ Introduction LTX-B
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## Nonrecycling walls simplify the tokamak



- Robustly flat ion, electron temperature profiles predicted
- No ∇T driven effects
  - No ion heat conduction losses
    - » Approaches thermal equilibrium in a confined plasma
  - No drive for the ITG, ETG
  - Bootstrap current driven only by ∇n
  - Particles transport heat.  $\tau_E = \tau_p$



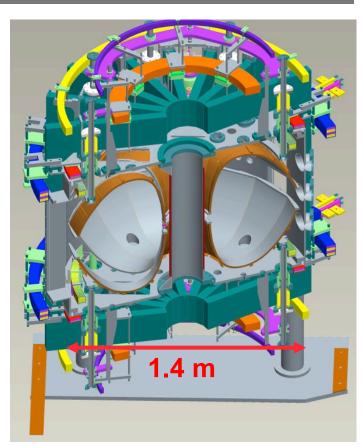
- Effect of recycling on edge temperature predicted by UEDGE modeling
  - > LLNL will study LTX-β SOL
- Edge power deposition profile strongly broadened
  - Hotter ions broader orbits
  - E×B convection, drifts ⇒ more broadening

### LTX and LTX-β



	LTX	LTX-β
А	1.6	1.6
$R_0$	40 cm	40 cm
а	26 cm	26 cm
B <sub>T</sub>	<1.7 kG	<3.4 kG
I <sub>p</sub>	<100 kA	<200 kA
P <sub>aux</sub>	0	700 kW
Pulse length	<50 msec	<100 msec

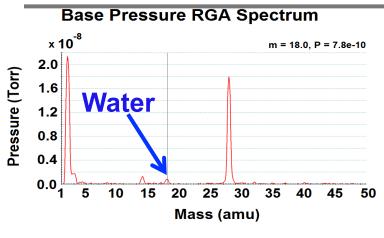
- High field-side limited by a conformal, high-Z wall
  - Not diverted
- Operated in hydrogen (gas puffing)
  - LTX: Fueled from the high field side midplane
    - LTX. I deled from the high field side inaplane



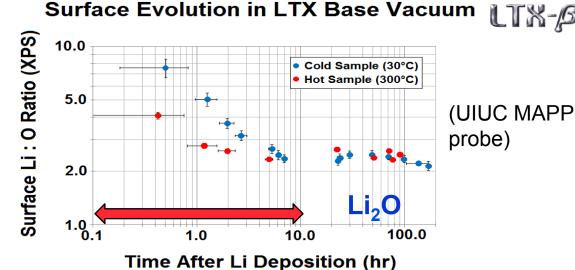
LTX-β: 35A neutral beam fueling, improved HFS puffing, topside SGI

## Very low partial pressure of water achieved in LTX

#### clean, metallic lithium PFCs



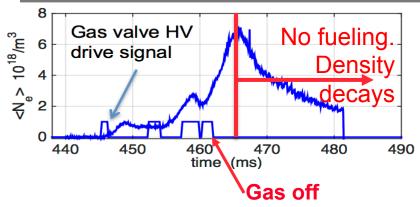
- Background water 5-9 × 10<sup>-10</sup> Torr in LTX
- Pumping speed doubled on LTX-β to 7,800 L/s
- Bakeout system upgraded



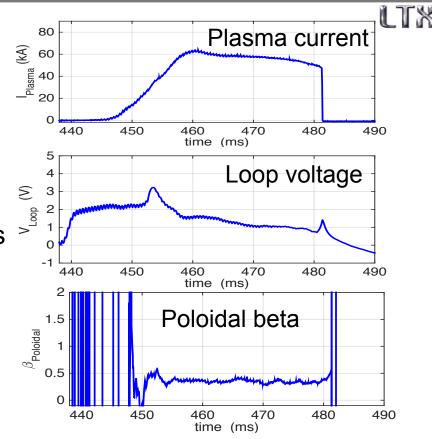
- X-ray Photoelectron Spectroscopy (XPS)
- Oxygen only impurity on lithium coating
- Li:O ratio initially high; asymptotes to 2:1 (Li<sub>2</sub>O)
- Expanded surface analysis on LTX-β –
   Princeton U. collaboration, U. Tennessee

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## Transient experiments eliminate edge neutral cooling

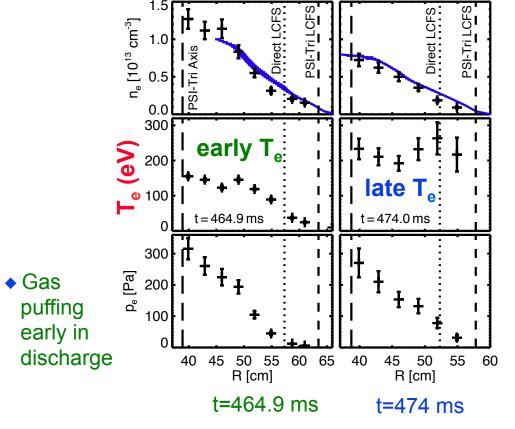


- Centerstack gas terminates at 462 ms
   ~3-4 ms to clear gas from nozzle
- Monitor T<sub>e</sub> evolution
- Thomson scattering time is stepped through the discharge
  - Dataset of 55 identical discharges
  - Average ~ 5 discharges/time

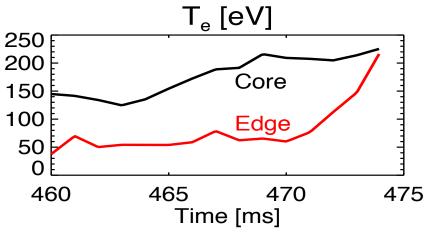


#### Control of recycling provides control over the **temperature** profile

Very low recycling allows completely flat temperature profiles to develop



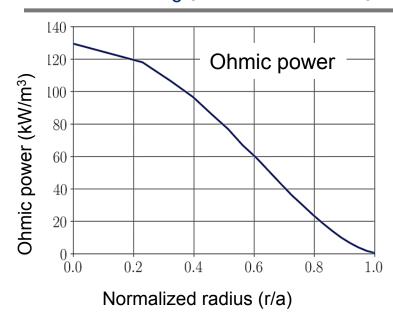
- Late in discharge:
  - > Lithium suppresses recycling
  - > No gas from puffing



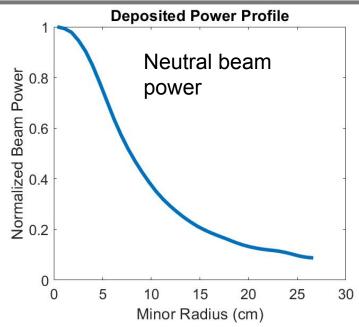
- >PSI-TRI equilibrium (U. Washington).
- >New equilibrium modeling from LiFusion
- >Thomson upgrades on LTX-β



#### Flat T<sub>e</sub> profile develops with peaked Ohmic power deposition



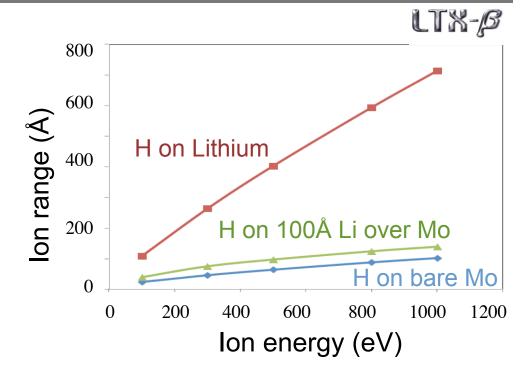
- ◆ T<sub>e</sub> profile is much broader than Ohmic power deposition profile
- NBI in LTX-β will test T profiles with ion heating



- Neutral beam heating profile will be more peaked
  - E<sub>beam</sub>>E<sub>critical</sub>
  - Electron and ion heating from NBI

## Recycling reduction robust with high edge T<sub>e</sub>

- Proton will penetrate 10's to 100 monolayers deep in metallic lithium coating
  - Buried in the solid
  - Recycling less dependent on near-surface conditions
  - Differs from NSTX case
- Large hydrogen inventory can be stored in the lithium wall
- Diffusivity of hydrogen, other impurities in *liquid* lithium will modify impurity profile



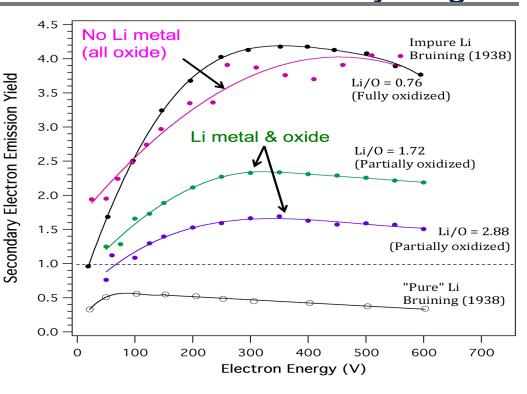
TRIM modeling by L. Buzi, Princeton U.

## Surface conditions still determine electron "recycling"

- Electrons are also "recycled" via secondary electron emission
  - Secondaries cool the edge.
     Simple estimate:

$$q_{pe} = (2kT_e + e\varphi_0) \frac{0.6n_e c_s}{(1 - \gamma_e)} - e\varphi_0 \gamma_e \frac{0.6n_e c_s}{(1 - \gamma_e)}$$

- Metallic lithium has low secondary electron emission
- SEE higher for contaminated lithium
  - LTX-β surface science will be studied by Princeton U., U. Tennessee
  - New lithium deposition systems between-shots coatings

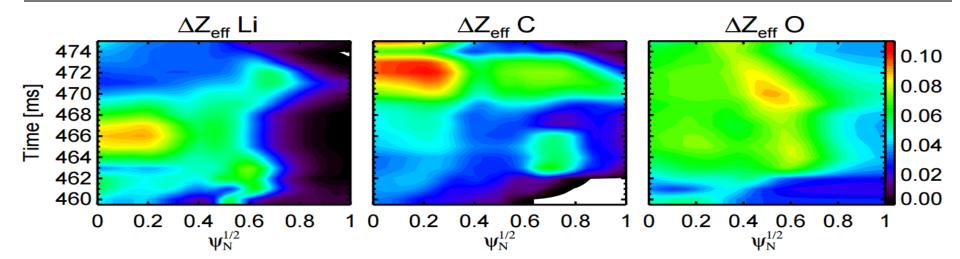


A. Capece, M. Patino, Y. Raitses, B. Koel, *Bull. Am. Phys. Soc., 2015.DPP.NO6.4* 



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## Low impurities -Lithium does not significantly dilute core plasma

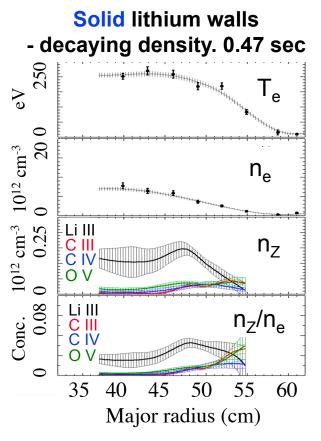


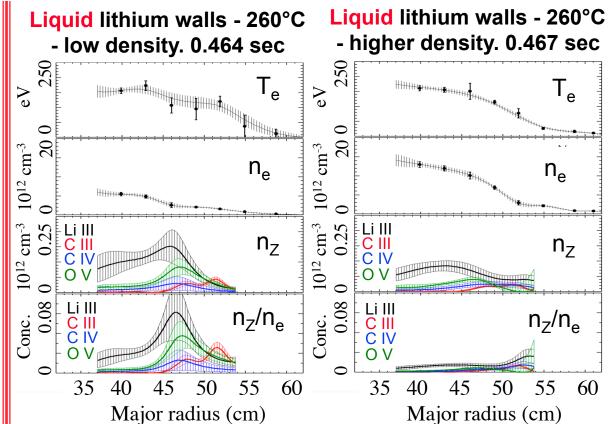
- Lithium impurity <2-3%</li>
  - Z<sub>effective</sub> remains below 1.2
  - Carbon remains a contributor to Z<sub>effective</sub>
- > Solid lithium PFCs



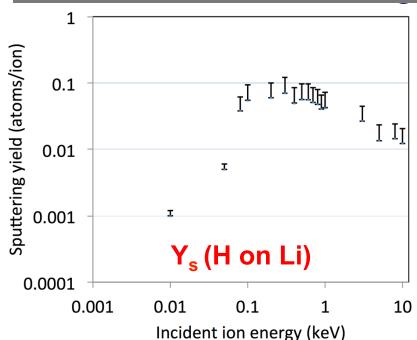


## Lithium, other impurities modest with liquid walls





## Lithium impurity influx will be further reduced at higher edge temperatures



TRIM modeling by L. Buzi, Princeton U.

- Hot edge during low recycling operation in LTX produced incident ion energies near the peak in the sputtering yield for H on lithium
  - Very low density edge
  - Ion flux very low
- Colder edge ions during liquid lithium operation 

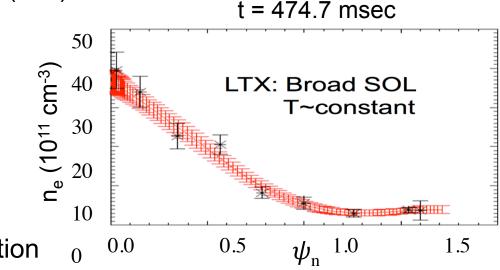
  □ lower yield
  - Higher edge density
  - Ion flux higher



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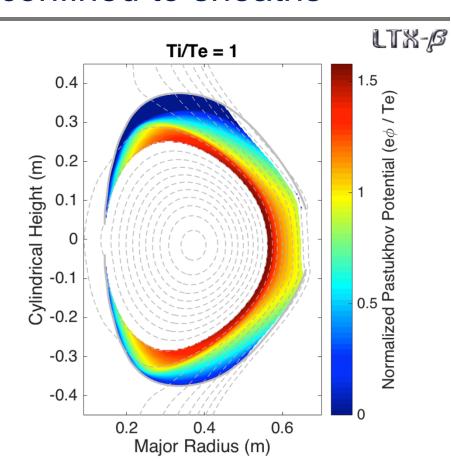
## Very low collisionality in LTX SOL

- ◆ Edge T<sub>e</sub> ~200 eV, T<sub>i</sub> ~40 70 eV (est.)
  - $n_e \sim 2-3 \times 10^{17} \text{ m}^{-3} \text{ (H)}$
- ◆  $L_{conn} \sim 5 10 \text{ m } (q_a \ge 5)$
- ◆ Neutral pressure < 10<sup>-5</sup> Torr
  - $-\lambda_{CX} \sim 1-2 \text{ km}$
- $\tau_{ee}$  ~ 400 µsec,  $\tau_{ii}$  ~ 1-2 msec
  - $L_{con}/C_{s} \sim 60 \, \mu sec \ll \tau_{ii}$
- ST: large trapped particle population
- Mirror ratio, LFS → HFS ~ 4
  - 80-90% SOL particles trapped



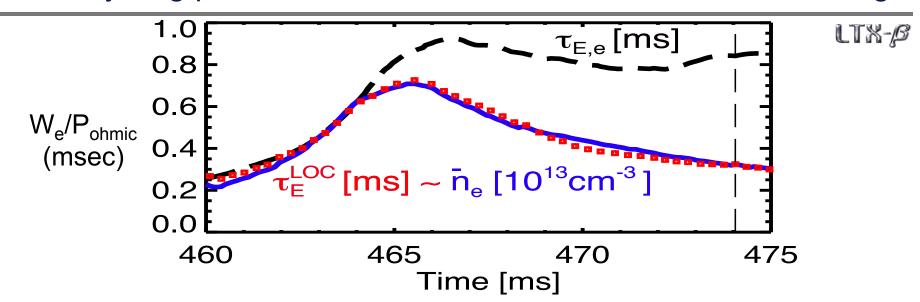
#### SOL electric fields not confined to sheaths

- Loss rate along SOL determined by ionion pitch angle scattering
  - SOL plasma mirror confined
- Pastukhov potential will develop [Nucl. Fusion 14 (1974)3]
  - $-\varphi_p$ ~0.6 0.8 kT<sub>e</sub> for LTX parameters
  - SOL electric field should strongly eject sputtered impurities
  - SOL diagnostics in LTX-β will include energy analyzers, langmuir and emissive probes
  - > LLNL, LiFusion will provide modeling support for LTX-β



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#### Low recycling plasma confinement exceeds neo-Alcator scaling

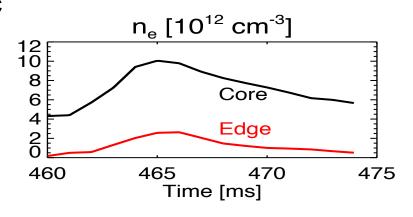


- Comparison is with neo-Alcator Linear Ohmic Confinement scaling
  - Appropriate for small tokamaks without auxiliary heating, like LTX
- Confinement time for LTX neglects ion stored energy; core T<sub>i</sub> uncertain
- Core transport and turbulence not diagnosed in LTX
- Kinetic measurements of stored energy emphasized in LTX-β

### Core particle confinement

- Core density decays with e-folding time of 15 – 20 msec
  - Edge density e-folding ~ 4.8 msec
- By comparison, τ<sub>i-e</sub> ~5-10 msec
  - Core electrons, ions expected to equilibrate
- Require core ion temperature measurement, local transport for accurate confinement estimates
- ORNL has installed CHERs diagnostic on LTX-β
- UCLA will contribute core fluctuation measurements (reflectometry)

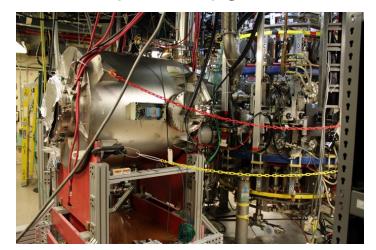


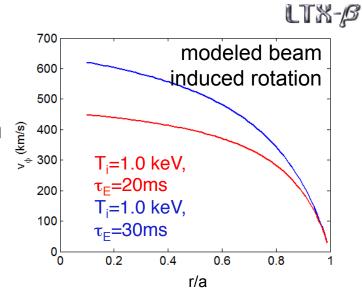


Evolution of edge and core n<sub>e</sub> in LTX

# LTX-β will combine gradient-free temperature profiles with sheared flow

- Two neutral beam injectors on loan from Tri - Alpha Technologies
  - One injector installed on LTX-β
  - 17-23 keV, 35A, hydrogen,  $R_{tan}$  = 23 cm
  - 5-8 msec pulse, upgrade to ~30 msec





- NBI support, fast ion physics: U. Wisconsin
- Momentum transport (& CHERs): ORNL

## Summary

- ◆ Liquid metal PFCs require engineering, physics development <a href="#">LTX P</a>
- Effects of lithium PFCs on transport, equilibrium, SOL not well understood
- ◆ Major effect on equilibrium  $\nabla T = 0$  demonstrated on LTX
  - Simplifies neoclassical transport in the tokamak
  - Effect on anomalous transport TBD
  - Extensibility to auxiliary heated systems TBD
- ◆ Impurity levels benign for T < 260 °C liquid lithium</p>
- SOL changes may be as significant as the core modifications
- LTX-β will examine a much broader parameter space than was accessible in LTX